UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 20, 1990

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P.O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. NPF-21 - WPPSS NUCLEAR PROJECT NO. 2 (TAC NO. 75983)

The U.S. Nuclear Regulatory Commission has issued the enclosed amendment to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated February 14, 1990 (G02-90-025).

This amendment revises Technical Specifications 3.3.7.5, "Accident Monitoring Instrumentation," and 3.4.2, "Safety/Relief Valves." Specifically the amendment revises the surveillance requirements for the two specifications by providing that the acoustic monitors for MS-RV-2C and 2D may be inoperable until the fifth refueling outage, currently scheduled to begin on or about April 13, 1990, or until the first forced outgage of sufficient duration to effect repair/replacement prior to that outage without applying the shutdown requirement. It also adds a footnote to Table 4.3.7.5-1, "Accident monitoring Instrumentation Surveillance Requirements," specifying that additional surveillances of tailpipe temperatures will be performed on the two affected valves. Prior to the amendment request, the specifications required that the monitors be returned to operability within seven days or that the plant be shut down.

WNP-2 entered action statements leading to plant shutdown on February 13, 1990 under the requirement of technical specification sections 3.3.7.5 and 3.4.1, and requested relief from those action statements to permit the unit to remain at power. Compensatory measures were proposed to ensure that safety is not impaired. Relief is granted by issuance of this amendment.

Because this amendment is needed to permit continued operation of the facility, this amendment is authorized on an emergency basis. The staff reviewed the circumstances associated with your request and concluded that you provided a sufficient basis for finding that the situation could not have been avoided by prior application. Therefore, in accordance with 10 CFR 50.91(a)(5), a valid emergency existed.

9003020032 900220
PDR ADOCK 05000397
P PDC

DF01
1/1

CP-1

A copy of the related safety evaluation supporting the amendment is enclosed. The notice of issuance and final determination of no significant hazards consideration and opportunity for hearing will be included with the Commission's biweekly Federal Register notices.

Sincerely,

Robert B. Samworth

Robert B. Samworth, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 75 to Facility Operating License No. NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

A copy of the related safety evaluation supporting the amendment is enclosed. The notice of issuance and final determination of no significant hazards consideration and opportunity for hearing will be included with the Commission's biweekly Federal Register notices.

Sincerely,

original signed by Robert Samworth

Robert B. Samworth, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 75 to Facility Operating License No. NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION

Docket File	D. Hagan
NRC & Local PDRs	E. Jordan
PD5 Reading	G. Hill (4)
C. Trammell	W. Jones
J. Zwolinski	J. Calvo
P. Shea	ACRS (10)
B. Samworth	GPA/PA
OGC	OC/LFMB
Region V (4)	PD 5 gray file

DRSP/PDV	DRSP/PDV	DST/SRXB	(A)DRSP/D/DRSP	OGC	D/DRSP(a)
PShea	RSamworth:sg	RCJones	CTrammell	JMoore	GHolahan
2/ /90	2/20/90	2/20/90	2/20/90	02/20/90	2/27/90

cc:
Mr. C. M. Powers
WNP-2 Plant Manager
Washington Public Power Supply System
P.O. Box 968, MD 927M
Richland, Washington 99352

Mr. G. E. Doupe, Esquire
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99532

Mr. Curtis Eschels, Chairman
Energy Facility Site Evaluation Council
Mail Stop PY-11
Olympia, Washington 98504

Mr. Alan G. Hosler, Licensing Manager
Washington Public Power Supply System
P. O. Box 968, MD 956B
Richland, Washington 99352

Mr. A. Lee Oxsen
Assistant Managing Director for Operations
Washington Public Power Supply System
P. O. Box 968, MD 1023
Richland, Washington 99352

Mr. Gary D. Bouchey, Director
Licensing and Assurance
Washington Public Power Supply System
P. O. Box 968, MD 280
Richland, Washington 99352

Regional Administrator, Region V
U.S. Nuclear Regulatory Commission
1450 Maria Lane, Suite 210
Walnut Creek, California 94596

Chairman
Benton County Board of Commissioners
Prosser, Washington 99350

Mr. Christian Bosted
U. S. Nuclear Regulatory Commission
P. O. Box 69
Richland, Washington 99352

Nicholas S. Reynolds, Esq.
Bishop, Cook, Purcell
& Reynolds
1400 L Street NW
Washington, D.C. 20005-3502

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Washington Public Power Supply System (the licensee), dated February 14, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9003020036 900220
PDR ADOCK 05000397
P PDC

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 75, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Gary M. Holahan

Gary M. Holahan, Director
Division of Reactor Projects - III
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 20, 1990

ENCLOSURE TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Also to be replaced are the following overleaf pages.

AMENDMENT PAGE	OVERLEAF PAGE
3/4 3-71	3/4 3-72
3/4 3-74	3/4 3-73
3/4 4-7	3/4 4-8

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ACTION
1. Reactor Vessel Pressure	2	1	1, 2	80
2. Reactor Vessel Water Level	2	1	1, 2	80
3. Suppression Chamber Water Level	2	1	1, 2	80
4. Suppression Chamber Water Temperature	2/sector	1/sector	1, 2	80
5. Suppression Chamber Air Temperature	2	1	1, 2	80
6. Drywell Pressure	2	1	1, 2	80
7. Drywell Air Temperature	2	1	1, 2	80
8. Drywell Oxygen Concentration	2	1	1, 2	80
9. Drywell Hydrogen Concentration	2	1	1, 2	80
10. Safety/Relief Valve Position Indicators	2/valve*	1/valve	1, 2	80
11. Suppression Chamber Pressure	2	1	1, 2	80
12. Condensate Storage Tank Level	2	1	1, 2	80
13. Main Steam Line Isolation Valve Leakage Control System Pressure	2	1	1, 2	80

*NOTE - The acoustic monitors for MS-RV-2C and 2D may be inoperable until the fifth Refueling Outage scheduled for No Later Than May 15, 1990, or until the first forced outage of sufficient duration to effect repair/replacement prior to that date without applying the shutdown requirement of Action 80.a

TABLE 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NUMBER OF CHANNELS	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ACTION
14. Neutron Flux:				
APRM	2	1	1, 2	80
IRM	2	1	1, 2	80
SRM	2	1	1, 2	80
15. RCIC Flow	1	1	1, 2	80
16. HPCS Flow	1	1	1, 2	80
17. LPCS Flow	1	1	1, 2	80
18. Standby Liquid Control System Flow	1	1	1, 2	80
19. Standby Liquid Control System Tank Level	1	1	1, 2	80
20. RHR Flow	1/loop	1/loop	1, 2	80
21. RHR Heat Exchanger Outlet Temperature	1/heat exchanger	1/heat exchanger	1, 2	80
22. Standby Service Water Flow	1/loop	1/loop	1, 2	80
23. Standby Service Water Spray Pond Temperature	2	1	1, 2	80
24. Post-Accident Sampling Containment Atmospheric Radiation Monitor	1	1	1, 2, 3	81
25. Emergency Ventilation Damper Position	2/duct	1/duct	1, 2	80
26. Standby Power and Other Energy Sources	2/source	1/source	1, 2	80
27. Primary Containment Valve Position	1/valve	1/line	1, 2	80
28. Primary Containment Gross Radiation Monitors#	2	1	1, 2, 3	81
29. Post-Accident Sampling Primary Coolant Radiation Monitor	1	1	1, 2, 3	81
30. Effluent Noble Gas Radiation Monitor#	1	1	1, 2, 3	81
31. Reactor Building Post LOCA Grab Sampler	1	1	1, 2, 3	81

#High range monitors.

Table 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

ACTION STATEMENTS

ACTION 80 -

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

ACTION 81 - With the number of OPERABLE accident monitoring instrumentation channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or:

a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and

b. In lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION	APPLICABLE OPERATIONAL CONDITIONS
1. Reactor Vessel Pressure	M	R	1, 2
2. Reactor Vessel Water Level	M	R	1, 2
3. Suppression Chamber Water Level	M	R	1, 2
4. Suppression Chamber Water Temperature	M	R	1, 2
5. Suppression Chamber Air Temperature	M	R	1, 2
6. Primary Containment Pressure	M	R	1, 2
7. Drywell Air Temperature	M	R	1, 2
8. Drywell Oxygen Concentration	M	R	1, 2
9. Drywell Hydrogen Concentration	M	Q	1, 2
10. Safety/Relief Valve Position Indicators	M*	R	1, 2
11. Suppression Chamber Pressure	M	R	1, 2
12. Condensate Storage Tank Level	M	R	1, 2
13. Main Steam Line Isolation Valve Leakage Control System Pressure	M	R	1, 2
14. Neutron Flux:			
APRM	M	R	1, 2
IRM	M	R	1, 2
SRM	M	R	1, 2
15. RCIC Flow	M	R	1, 2
16. HPCS Flow	M	R	1, 2
17. LPCS Flow	M	R	1, 2

*Surveillance of the OPERABLE Tailpipe Temperature instrument channels for SRV MS-RV-2C and 2D will be performed daily until the acoustic monitor for that valve is once again declared OPERABLE.

3/4.4.2 SAFETY/RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.4.2 The safety valve function of at least 12 of the following reactor coolant system safety/relief valves shall be OPERABLE with the specified code safety valve function lift settings:*

2 safety/relief valves @ 1150 psig +1%/-3%
4 safety/relief valves @ 1175 psig +1%/-3%
4 safety/relief valves @ 1185 psig +1%/-3%
4 safety/relief valves @ 1195 psig +1%/-3%
4 safety/relief valves @ 1205 psig +1%/-3%

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

a. With the safety valve function of one or more of the above required safety/relief valves inoperable, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.

b. With one or more safety/relief valves stuck open, provided that suppression pool average water temperature is less than 90°F, close the stuck open safety/relief valve(s); if unable to close the open valve(s) within 2 minutes or if suppression pool average water temperature is 110°F or greater, place the reactor mode switch in the Shutdown position.

c. With one or more safety/relief valve acoustic monitors inoperable, restore the inoperable monitor(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours. ***

SURVEILLANCE REQUIREMENTS

4.4.2 The acoustic monitor for each safety/relief valve shall be demonstrated OPERABLE by performance of a:

a. CHANNEL CHECK at least once per 31 days, and a

b. CHANNEL CALIBRATION at least once per 18 months.**

---

*The lift setting pressure shall correspond to ambient conditions of the valves at nominal operating temperatures and pressures.

**The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure is adequate to perform the test.

***The acoustic monitors for MS-RV-2C and 2D may be inoperable until the fifth Refueling Outage scheduled for No Later Than May 15, 1990, or until the first forced outage of sufficient duration to effect repair/replacement prior to that date without applying the shutdown requirement.

3/4.4.3 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.3.1 The following reactor coolant system leakage detection systems shall be OPERABLE:

a. The primary containment atmosphere gaseous radioactivity monitoring system,

b. The primary containment sump flow monitoring system, and

c. The primary containment atmosphere particulate radioactivity monitoring system.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

With only two of the above required leakage detection systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required gaseous and/or particulate radioactive monitoring system is inoperable; otherwise, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

4.4.3.1 The reactor coolant system leakage detection systems shall be demonstrated OPERABLE by:

a. Primary containment atmosphere particulate and gaseous monitoring systems-performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

b. Primary containment sump flow monitoring system-performance of a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION TEST at least once per 18 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. NPF-21

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated February 14, 1990 (G02-90-025) Washington Public Power Supply System proposed certain changes to the Technical Specifications for Nuclear Project No. 2. Specifically the Supply System requested that notes be added to Specification 3.4.2, Safety Relief Valves and to Tables 3.3.7.5-1, Accident Monitoring Instrumentation, and 4.3.7.5-1 Accident Monitoring Instrumentation Surveillance Requirements to allow a single acoustic monitor on each of two SRVs to be inoperable until the plant shuts down for its next scheduled refueling outage or until the first forced outage of sufficient duration to effect repair, whichever occurs first. The note on Table 4.3.7.5-1 specified additional compensatory sampling of the tailpipe temperatures of the two affected valves.

Action c for Specification 3.4.2, Safety Relief Valve and action 80.a for Technical Specification Table 3.3.7.5-1, Accident Monitoring Instrumentation, require that the plant be shutdown if an inoperable safety/relief valve indicator channel is not restored to operable status within seven (7) days. Due to apparent failure of the Acoustic Monitor for MS-RV-2D, the Supply System entered the seven day LCO at 5:27 p.m. PST on February 13, 1990. It has been determined that the plant will be required to be shutdown and cooled down in order to repair/replace this acoustic monitor, unless an amendment is granted to allow the Supply System to continue to operate until either the next forced outage or the next scheduled outage. The licensee is scheduled to shutdown for a refueling outage on or about April 13, 1990.

The acoustic monitor for MS-RV-2C is exhibiting behavior similar to that observed on the MS-RV-2D acoustic monitor prior to its failure. From these similar observations it has been concluded that a similar failure of the acoustic monitor for MS-RV-2C is likely. For this reason relief is requested for both acoustic monitors.

2.0 EVALUATION

The requirement for operability of the accident monitoring instrumentation is based on the need to ensure that sufficient information is available on selected plant parameters (e.g., SRV position indication) to monitor and assess important variables following an accident. TMI Action Plant

9003020041 900220
PDR ADOCK 05000397
P PDC

Item II.D.3 "Direct Indication of Relief and Safety-Valve Position" requires that "reactor coolant system relief and safety valves shall be provided with a positive indication in the control room derived from a reliable valve-position detection device or a reliable indication of flow in the discharge pipe." The technical specifications for WNP-2 require two instrumentation channels for providing this information on valve position. One channel utilizes an acoustic monitor. The second channel utilizes thermocouples to detect a temperature increase indicative of flow past the valve.

The redundancy contributes to reduction in accident risk. The loss of either channel on one or more SRVs does not prevent accurate determination of the position of the associated SRV(s). Total loss of position indication would allow an SRV to be stuck in the open position without this being known immediately to the operator. As discussed in the safety analysis of this event (WNP-2 FSAR 15.1.4), the operator response to this event is triggered by a suppression pool temperature alarm not by an "open" alarm from the SRV position indication instruments.

The mitigating actions are to locate and attempt to close the open SRV and establish suppression pool cooling within 20 minutes. As discussed in the FSAR analysis, even if the valve fails to close (worst case) the consequences of the event are mild. Hence the failure of an acoustic monitor causing an operator to review other instrumentation (as listed below) to determine which valve is open does not increase the severity of the transient.

The loss of position indication for one or more of the eighteen (18) Safety/ Relief Valves does not reduce the capability of the SRV to perform its intended function, nor does it prevent accurate determination of the position of the associated SRV.

Similar relief requests for inoperable relief valve acoustic monitors were made by the licensee on October 13, 1987 and on July 25, 1988. Subsequently the Supply System has made modifications to the acoustic monitors and changed procedures to increase the reliability of the monitors. This effort is ongoing. A design change is in progress that would allow accessibility, without plant shutdown, of the suspected failed component pending successful environmental qualification. The signal amplifiers (suspected component) and associated cabling for these monitors with the exception of 2D have been replaced. The monitor MSRV-2D is scheduled to be replaced at the next outage. Further a Vendor representative site visit has been requested to assist in identifying additional enhancements to system reliability. The Supply System has been and is continuously working to improve this system.

The following mitigating and compensatory factors provide assurance that the valve position is reliably known:

1. Tail pipe temperature is monitored and recorded. An increase in temperature would indicate that the valve is open, and steam was entering the suppression pool via the tail pipe. This indication has remained capable of indicating valve actuation since the failed monitor was identified. Channel checks of the temperature recorder are currently performed monthly per LCO 3/4.3.7.5. Until the acoustic

monitors for MSRV-RV-2C and 2D are declared operable, the tail pipe temperature surveillance will be performed daily instead of monthly. A control room annunciator alarms on high tail pipe temperature (greater than 250oF). An annunciator response procedure addressing the tailpipe temperature alarm will be revised to uniquely identify appropriate actions for MS-RV-2C and 2D.

2. Suppression Pool temperature indication is available, and is set to alarm at 85oF. An increase in suppression pool temperature would indicate an open SRV. This parameter will be monitored on a daily basis.

3. Suppression Pool level indication is available, and is set to alarm at +0.5"/-1" of Normal Level (466'3"). An increase in suppression pool level would indicate an open SRV. This parameter will be monitored on a daily basis.

4. Cross talk due to noise pickup from acoustic monitors on adjacent SRVs will indicate flow through the SRV with the failed monitor but will not alarm. This capability has been demonstrated to be a reliable diagnostic tool on several occasions in the past.

5. Other plant parameters are affected by an SRV actuation and are available as confirmation. Examples are main turbine governor valve position indication change, generator output change, main turbine steam flow change, steam/feedwater flow mismatch and the resultant reactor pressure perturbation.

Other indicators provide adequate feedback for ADS (reactor pressure) and SRV operation, and Alternate Shutdown Cooling operation (reactor pressure/temperature) if they are required. Additionally, MS-RV-2C and 2D are not controlled on either of the remote shutdown panels and are not ADS valves.

With the compensatory measures proposed by the licensee, the staff finds the safety significance to be small and the proposed temporary amendment to be acceptable.

3.0 EMERGENCY CIRCUMSTANCES

Failure of these acoustic monitors has only recently been identified. The licensee declared the monitor for valve MS-RV-2D INOPERABLE AT 5:27 p.m., February 13, 1990. It was not possible to anticipate this event and submit this request in a more timely manner. The licensee has determined that repair will require that the reactor be shut down and cooled down in order to permit drywell entry. Based on the above, the licensee requested the emergency temporary change to the technical specifications to allow continued operation until the next outage of sufficient duration to effect the necessary repairs. Absent this amendment, the Supply System would be required to unnecessarily shutdown by February 20, 1990. The next scheduled outage for refueling will start on or about April 13, 1990.

The staff reviewed the circumstances associated with your request and concluded that you provided a sufficient basis for finding that the situation could not have been avoided by prior application. Therefore, in accordance with 10 CFR 50.91(a)(5), a valid emergency existed.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in requirements with respect to the use or installation of a facility component located within the restricted area as defined in 10 CFR Part 20 and in surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that a license amendment involves no significant hazards considerations if operation of the facility in accordance with the amendment would not:

1. Involve a significant increase in the probability or the consequences of any accident previously evaluated; or

2. Create the possibility of a new or different kind of accident from any accident previously evaluated; or

3. Involve a significant reduction in a margin of safety.

The amendment has been evaluated against these standards in 10 CFR 50.92. A discussion of these standards as they relate to the amendment request follows:

1. The change does not involve a significant increase in the probability or consequence of an accident previously evaluated because the SRV position indication channels are not assumed to function in the initiation of an analyzed accident. The inoperability of these indication channels does not affect ADS operation of the SRVs. The analysis for an inadvertent opening of an SRV (FSAR Section 15.1.4) assumes the function of these alarm-only instrument channels for the purpose of having the operator assess the need for commencing suppression pool cooling with RHR. As discussed above, the operator has many diverse indications available to indicate the need for commencing suppression pool cooling as a result of an open SRV and the SRV position indication is not the primary indications. Loss of an SRV position indication channel will not adversely affect the operator's ability to respond to this event as assumed in the analysis. The proposed change affects only the operability of the SRV position indication and does not affect automatic or manual actuation of the SRV. MS-RV-2C and 2D are not ADS valves nor are

they controlled from either of the remote shutdown panels (Appendix R). Therefore, this change will not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The change does not create the possibility of a new or different kind of accident from any accident previously because SRV operation, including the ADS function, remains unaffected. No new modes of operation of any equipment result due to this change. Sufficient diverse indication remains available to adequately determine whether an SRV is inadvertently open, therefore this change will not result in a failure to assess the need for suppression pool cooling. This change will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The change does not create a significant reduction in a margin of safety because, as discussed above, the operator has many diverse indications available to indicate the need for commencing suppression pool cooling. Loss of an SRV position indication channel will not adversely affect the operator's ability to respond to this event as assumed in the analysis. The additional surveillances to monitor the suppression pool temperature while operation continues with an inoperable channel will compensate for the loss of position indication channel. Therefore, this change will not involve a significant reduction in the margin of safety.

Accordingly, the Commission has determined that this amendment involves no significant hazards consideration.

6.0 CONTACT WITH STATE OFFICIAL

In accordance with 10 CFR 50.91, the licensee provided the State of Washington with a copy of its February 14, 1990 letter. The NRC staff advised the Washington Energy Facility Siting Council of the final determination of no significant hazards considerations by telephone on February 20, 1990. The State of Washington did not have any comment on this determination.

7.0 CONCLUSION

In summary, based on the assertion that no significant hazard is created by the proposed amendment and that the proposed compensatory actions provide reliable indication of SRV position, approval of the proposed amendment does not represent an undue risk to the health and safety of the public.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert B. Samworth

Dated: February 20, 1990