

Docket No. 50-397

April 2, 1990

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Mr. G. C. Sorensen, Manager  
Regulatory Programs  
Washington Public Power Supply System  
P.O. Box 968  
3000 George Washington Way  
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE  
NO. NPF-21 - WPPSS NUCLEAR PROJECT NO. 2 (TAC NO. 73783)

The U.S. Nuclear Regulatory Commission has issued the enclosed amendment to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated June 23, 1989 (G02-89-114).

This amendment revises Technical Specification 3/4.3.7.5, Accident Monitoring Instrumentation, by eliminating the requirement for two accident monitoring instruments. Specifically, instrument 24, Post-Accident Sampling Containment Atmospheric Radiation Monitor, and instrument 29, Post-Accident Sampling Primary Coolant Radiation Monitor are removed from Table 3.3.7.5-1, Accident Monitoring Instrumentation. The surveillance requirements for these two instruments are also eliminated by removing the instruments from Table 4.3.7.5-1, Accident Monitoring Instrumentation Surveillance Requirements.

A copy of the related safety evaluation supporting the amendment is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY R. SAMWORTH

Robert B. Samworth, Senior Project Manager  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

**Enclosures:**

1. Amendment No. 79 to Facility  
Operating License No. NPF-21
2. Safety Evaluation

cc w/enclosures:

See next page

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PShea  
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OGC  
RBachmann  
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DRSP:PD5  
CMTrammell  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 2, 1990

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Regulatory Programs  
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A copy of the related safety evaluation supporting the amendment is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Robert B. Samworth", is positioned below the "Sincerely," text.

Robert B. Samworth, Senior Project Manager  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 79 to Facility  
Operating License No. NPF-21
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. G. C. Sorensen

WPPSS Nuclear Project No. 2  
(WNP-2)

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79  
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Washington Public Power Supply System (the licensee), dated June 23, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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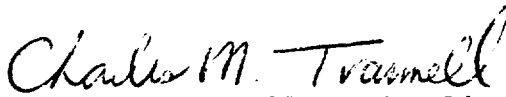
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 79 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Charles M. Trammell, Acting Director  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 2, 1990

ENCLOSURE TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Also to be replaced are the following overleaf pages.

AMENDMENT PAGE

3/4 3-72

3/4 3-75

OVERLEAF PAGE

3/4 3-71

3/4 3-76

TABLE 3.3.7.5-1  
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1, 2	80
2. Reactor Vessel Water Level	2	1	1, 2	80
3. Suppression Chamber Water Level	2	1	1, 2	80
4. Suppression Chamber Water Temperature	2/sector	1/sector	1, 2	80
5. Suppression Chamber Air Temperature	2	1	1, 2	80
6. Drywell Pressure	2	1	1, 2	80
7. Drywell Air Temperature	2	1	1, 2	80
8. Drywell Oxygen Concentration	2	1	1, 2	80
9. Drywell Hydrogen Concentration	2	1	1, 2	80
10. Safety/Relief Valve Position Indicators	2/valve*	1/valve	1, 2	80
11. Suppression Chamber Pressure	2	1	1, 2	80
12. Condensate Storage Tank Level	2	1	1, 2	80
13. Main Steam Line Isolation Valve Leakage Control System Pressure	2	1	1, 2	80

\*NOTE - The acoustic monitors may be inoperable until the fifth Refueling Outage scheduled for No Later Than May 15, 1990 without applying the shutdown requirement of Action 80.a.

TABLE 3.3.7.5-1 (Continued)  
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
14. Neutron Flux:				
APRM	2	1	1, 2	80
IRM	2	1	1, 2	80
SRM	2	1	1, 2	80
15. RCIC Flow	1	1	1, 2	80
16. HPCS Flow	1	1	1, 2	80
17. LPCS Flow	1	1	1, 2	80
18. Standby Liquid Control System Flow	1	1	1, 2	80
19. Standby Liquid Control System Tank Level	1	1	1, 2	80
20. RHR Flow	1/loop	1/loop	1, 2	80
21. RHR Heat Exchanger Outlet Temperature	1/heat exchanger	1/heat exchanger	1, 2	80
22. Standby Service Water Flow	1/loop	1/loop	1, 2	80
23. Standby Service Water Spray Pond Temperature	2	1	1, 2	80
24.				
25. Emergency Ventilation Damper Position	2/duct	1/duct	1, 2	80
26. Standby Power and Other Energy Sources	2/source	1/source	1, 2	80
27. Primary Containment Valve Position	1/valve	1/line	1, 2	80
28. Primary Containment Gross Radiation Monitors#	2	1	1, 2, 3	81
29.				
30. Effluent Noble Gas Radiation Monitor#	1	1	1, 2, 3	81
31. Reactor Building Post LOCA Grab Sampler	1	1	1, 2, 3	81

#High range monitors.



TABLE 4.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>
18. Standby Liquid Control System Flow	M	R	1, 2
19. Standby Liquid Control System Tank Level	M	R	1, 2
20. RHR Flow	M	R	1, 2
21. RHR Heat Exchanger Outlet Temperature	M	R	1, 2
22. Standby Service Water Flow	M	R	1, 2
23. Standby Service Water Spray Pond Temperature	M	R	1, 2
24.			
25. Emergency Ventilation Damper Position	M	R	1, 2
26. Standby Power and Other Energy Sources	M	R	1, 2
27. Primary Containment Valve Position	M	R	1, 2
28. Primary Containment Gross Radiation Monitors	M	R*	1, 2, 3
29.			
30. Effluent Noble Gas Radiation Monitor#	M	R	1, 2, 3
31. Reactor Building Post LOCA Grab Sampler	M	R	1, 2, 3

TABLE NOTATION

\*CHANNEL CALIBRATION shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/h and a one point calibration check of the detector below 10 R/h with an installed or portable gamma source.

## INSTRUMENTATION

### SOURCE RANGE MONITORS

#### LIMITING CONDITION FOR OPERATION

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3.3.7.6 At least the following source range monitor channels shall be OPERABLE:

- a. In OPERATIONAL CONDITION 2\*, three.
- b. In OPERATIONAL CONDITIONS 3 and 4, two.

APPLICABILITY: OPERATIONAL CONDITIONS 2\*, 3, and 4.

#### ACTION:

- a. In OPERATIONAL CONDITION 2\* with one of the above required source range monitor channels inoperable, restore at least 3 source range monitor channels to OPERABLE status within 4 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- b. In OPERATIONAL CONDITION 3 or 4 with one or more of the above required source range monitor channels inoperable, verify all insertable control rods to be inserted in the core and lock the reactor mode switch in the Shutdown position within 1 hour.

#### SURVEILLANCE REQUIREMENTS

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4.3.7.6 Each of the above required source range monitor channels shall be demonstrated OPERABLE by:

- a. Performance of a:
  1. CHANNEL CHECK at least once per:
    - a) 12 hours in CONDITION 2\*, and
    - b) 24 hours in CONDITION 3 or 4.
  2. CHANNEL CALIBRATION\*\* at least once per 18 months.
- b. Performance of a CHANNEL FUNCTIONAL TEST:
  1. Within 24 hours prior to moving the reactor mode switch from the Shutdown position, if not performed within the previous 7 days, and
  2. At least once per 31 days.
- c. Verifying, prior to withdrawal of control rods, that the SRM count rate is at least 0.5 cps# with the detector fully inserted.

\*With IRM's on range 2 or below.

\*\*Neutron detectors may be excluded from CHANNEL CALIBRATION.

#Provided signal-to-noise ratio is  $\geq 2$ . Otherwise, 3 cps.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. NPF-21  
WASHINGTON PUBLIC POWER SUPPLY SYSTEM  
NUCLEAR PROJECT NO. 2  
DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated June 23, 1989, the Washington Public Power Supply Systems, licensee for Washington Nuclear Plant Unit No. 2 (WNP-2), submitted a request for license amendment to Facility Operating License NPF-21 to delete two radiation monitors from the WNP-2 Technical Specifications (TS). These two monitors are Post-Accident Sampling Containment Atmospheric (PASCA) Radiation Monitor and Post-Accident Sampling Primary Coolant (PASPC) Radiation Monitor. Both monitors are specified in WNP-2 TS Tables 3.3.7.5-1 and 4.3.7.5-1, Accident Monitoring Instrumentation.

2.0 EVALUATION

2.1 Post-Accident Sampling Containment Atmospheric Radiation Monitor, TS Tables 3.3.7.5-1 and 4.3.7.5-1, Accident Monitoring Instrumentation.

The PASCA radiation monitor is installed at WNP-2, adjacent to the post accident containment atmosphere iodine sampling cartridge, and monitors the iodine activity buildup in the sample cartridge during and following an accident. This monitor neither functions nor actuates to detect or mitigate an accident.

The monitor only provides useful additional information to the plant operator following an accident. The licensee proposes not to remove the monitor from service but merely to delete it from the WNP-2 TS. The WNP-2 operating procedure, PPM-12.10.1 "Post Accident Sampling System Operation (Rev. 5)," dated March 11, 1988, requires that the plant operators utilize a separate portable radiation monitor (in addition to the PASCA radiation monitor) to estimate iodine activity buildup in the sample cartridge. The staff, therefore, finds that the request to delete the radiation monitor from the TS is acceptable.

2.2 Post-Accident Sampling Primary Coolant Radiation Monitor, Technical Specification Tables 3.3.7.5-1 and 4.3.7.5-1, Accident Monitoring Instrumentation.

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The PASPC radiation monitor detects liquid sample flow of the primary coolant to provide immediate assessment of the activity level following an accident and to provide information about the effectiveness of the demineralized water flushing of the sample line following sample collection. This monitor neither functions nor actuates to detect or mitigate an accident. However, the monitor does provide additional information to the plant operator.

The licensee proposes not to remove the PASPC monitor but merely to delete it from the WNP-2 TS. The WNP-2 TS requires in Section 6.8.4(c) a program that will ensure the capability to obtain a reactor coolant sample in accordance with the requirements specified in NUREG-0737, Item II.B.3. The licensee stated that these requirements are met with plant operating procedure, PPM-1.11.6, "Post-Accident Sampling System (Rev. 2)," dated March 10, 1988. Therefore, the staff finds that the request to delete the monitor from the TS is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation and use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONTACT WITH STATE OFFICIAL

The Commission made a proposed determination that the amendment involves no significant hazards consideration (54 FR 42866, October 18, 1989) and consulted with the State of Washington. No public comments were received, and by letter dated January 17, 1990 the State of Washington advised that they do not have any comment.

### 5.0 CONCLUSION

On the basis of this safety evaluation, the staff concludes that the proposed license amendment to delete two radiation monitors from the WNP-2 TS is acceptable. The bases for our acceptance are that neither the Standard Technical Specifications for Boiling Water Reactors (NUREG-0123, Rev. 3) dated Fall 1980 nor TMI Action Item II.B.3 (NUREG-0737) Post Accident Sampling Capability, specifically requires these monitors to be provided in the operation of boiling water reactors.

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Lee

Dated: April 2, 1990