

29 APR 1987

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

Subject: ISSUANCE OF EXEMPTION TO A PROVISION OF APPENDIX J AND
AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. NPF-21
WPPSS NUCLEAR PROJECT NO. 2 (TAC NO. 60740)

The U.S. Nuclear Regulatory Commission has issued the enclosed exemption from a provision of Appendix J to 10 CFR Part 50, and the enclosed Amendment No. 41 to Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This exemption and the amendment of the license are issued in response to your application dated January 31, 1986.

The exemption relieves the licensee from the requirement that Type C leak testing of containment valves be done during each reactor shutdown for refueling. Both the exemption and the amendment to the Technical Specifications extend the maximum interval between performances of both Type B and Type C tests from 24 to 27 months.

The safety evaluation for these licensing actions is provided in the exemption and is incorporated by reference in the Amendment. Also enclosed is a copy of a related notice of environmental assessment and finding of no significant impact which was published in the Federal Register on April 3, 1987. The notice of issuance of the amendment will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Original signed by
Gary M. Holahan for

Dennis M. Crutchfield, Director
Division of Reactor Projects - III/IV/V
& Special Projects

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Enclosures:

1. Exemption
2. Amendment No. 41 to Facility Operating License NPF-21
3. Safety Evaluation
4. Notice of Environmental Assessment

cc w/enclosures:
See next page

*See Previous Concurrence

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BSamworth

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*OGC

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DCrutchfield
4/29/87

Docket No. 50-397

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Washington Public Power Supply System
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3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

Subject: Issuance of Exemption to a Provision of Appendix J and
Amendment No. 41 to Facility Operating License No. NPF-21
WPPSS Nuclear Project No. 2

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Sincerely,

Dennis Crutchfield, Director
Division of Reactor Projects - III/IV/V
& Special Projects

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Docket No. 50-397

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Regulatory Programs
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

Subject: Issuance of Exemption to a Provision of Appendix J and
Amendment No. 41 to Facility Operating License No. NPF-21
WPPSS Nuclear Project No. 2

The U.S. Nuclear Regulatory Commission has issued the enclosed exemption from a provision of Appendix J to 10 CFR Part 50, and the enclosed Amendment No. 41 to Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This exemption and the amendment of the license are issued in response to your application dated January 17, 1986.

The exemption relieves the licensee from the requirement that Type C leak testing of containment valves be done during each reactor shutdown for refueling. Both the exemption and the amendment to the Technical Specifications extend the maximum interval between performances of both Type B and Type C tests from 24 to 27 months.

The safety evaluation for these licensing actions is provided in the exemption and is incorporated by reference in the Amendment. Also enclosed is a copy of a related notice of environmental assessment and finding of no significant impact which was published in the Federal Register on April 3, 1987. The notice of issuance of the amendment will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Dennis Crutchfield, Director
Division of Reactor Projects - III/IV/V
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4/ /87

Mr. G. C. Sorensen, Manager
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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of Appendix J defines "Type C Tests", in part, as those intended to measure containment isolation valve leakage rates. Paragraph III.D.3 of Appendix J states, in part, "Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than two years." By letter dated January 31, 1986, the licensee requested an exemption from the requirement for the Type C testing during each reactor refueling shutdown, and an extension of the maximum interval from 24 to 27 months for both Type B and Type C testing. The Supply System is constrained by the relative abundance of hydro-electric power in the Pacific Northwest during the spring to shutdown every year at that time. Refueling is required during each of these shutdowns to ensure continuous operation throughout the remainder of the year when the availability of the nuclear power is critical. Thus the regulation and this weather related peculiarity require the Supply System to perform Type C testing on all specified valves every year. This exemption will permit Type C testing of each affected valve on a two-year cycle with approximately half of Type C valves tested each year during the spring refueling outage. In addition, the maximum allowable interval before retesting is extended to 27 months to allow for variations in the weather related length of the actual refueling cycle from year to year.

III.

To support the requested exemptions from the requirements of 10 CFR 50, Appendix J, the licensee provided the following rationale:

- A. The Bonneville Power Administration directs the licensee to refuel WNP-2 on a yearly basis, ideally coinciding with the peak period of hydro-electric generation. Strict compliance with Appendix J would require yearly testing of all Type C barriers. This frequent testing

consumes more resources than appropriate, and is not in keeping with ALARA considerations.

- B. To utilize fully all sources of power production, WNP-2 refueling outages are established based on regional weather as indicated in Section II, above. This approach in scheduling results in either an early or late outage depending on the weather conditions for that particular year. The added variability of the refueling schedule, therefore, should be considered when establishing the allowed maximum interval between tests.
- C. The intent of Appendix J is to leak test during a refueling outage, but not to require a shutdown solely for local leak rate testing. Since forecasting the exact date for any given refueling outage is not exact, the three-month allowance to the two-year test interval would avoid unnecessary plant shutdowns at periods of greatest need.

The staff has reviewed the exemption requests and the associated justification, and believes that the technical rationale has merit. Based on a series of discussions, augmentations, and clarifications to the original request, the licensee has modified the original proposal via a series of letters dated April 11 and July 22, 1986, and January 9, February 11, March 4 and April 7, 1987. The program now consists of the following elements:

1. All barriers that are to be tested under the requirements of 10 CFR 50, Appendix J, have been placed into three groups. In one group are the valves that will be tested each refueling outage. The remaining barriers have been divided into two approximately equal groups. These two groups are the barriers that will be alternately tested in

- two consecutive refueling outages. Testing will be done in the "as found" condition prior to any maintenance or repair of the barrier.
2. All containment barriers tested under Appendix J will be at intervals not to exceed 27 months. Nominally, the maximum testing interval will be 24 months.
 3. The testing frequency of the following valves/penetrations will not be affected by this exemption or amendment.
 - (a) Main Steam Isolation Valves (tested at an interval not to exceed 18 months);
 - (b) Containment Purge Supply and Exhaust Valves (6 months);
 - (c) Personnel Airlock (6 months); and
 - (d) Reactor Feedwater Check Valves used for Containment Isolation (each refueling).
 4. For valves/penetrations which are to be tested every other refueling outage, the licensee will apply acceptance leakage criteria to the test results in addition to the requirements of Appendix J. The licensee's criteria are described below.

For valves, the leakage criterion is based on permissible leakage rates established by the ASME Code, Section XI, Article 3426. The methodology determines the leakage limit as a function of valve diameter using the following relationship for valves 10 inches in diameter or less:

$$L = 7.5 D$$

where:

L = maximum permissible leakage rate, standard cubic feet per day. (scf/day); and

D = valve diameter (inches).

For valves greater than 10 inches, the allowable leakage limit will be 60 percent of the value obtained using the above formula. At the present time, WNP-2 has 346 valves in the group that will be assigned alternate yearly testing. The valve diameters range from 0.5 to 24 inches. Using the above criteria, the leakage limit in terms of La would vary between .001 La and .03 La. Assuming that every valve leaked at its limits, the maximum cumulative leakage for these valves would be 1.0 La. This total includes the valve leakage for valves greater than 10 inches using 60 percent of the calculated value.

5. For the Type B penetrations, the licensee proposes to set the acceptance criterion for leakage at 50 standard cubic centimeters per minute (sccm) per penetration. These Type B penetrations include electrical penetrations, drywell head, equipment hatch, inspection ports, etc., but do not include airlocks. This leakage criterion is based on past experience of the licensee.
6. During each refueling outage, the combined Type B and C leakage will be computed based on "as left" leakage upon the completion of the current leakage tests. To obtain the total leakage, the "as left" leakage values for valves not tested during the current testing schedule will be used.
7. The individual barrier leakage criteria, if not met, will result in two actions. First, the barrier will be included in the group to be tested during the following refueling outage. Second, the barrier will be considered for repair during the current outage. The decision to repair will be made on a case-by-case basis.

8. Following each operating cycle refueling outage and prior to restarts the total "as left" Types B & C leakage rates shall not exceed 0.5 La (in lieu of the 0.6 La required by Appendix J). Additionally, if at any time during an operating cycle, the "as left" leakage total following maintenance exceeds 0.5 La, all Type B & C barriers will be tested during the next shutdown for refueling.
9. For the Type B and Type C testing programs, the reporting requirements of Appendix J will be augmented to include the information associated with the unique aspects of the WNP-2 program. In particular, the report will note the acceptance leakage criteria for each barrier as well as the barriers that failed the test criteria and consequently will be tested during the following refueling outage. Also included in the report will be the list of valves/penetrations not tested during the outage but that are scheduled to be tested during the next refueling outage.

To support their program, the licensee has reviewed the test data obtained to date from the WNP-2 plant. Three previous tests have shown that 70% of the isolation valves tested in the "as found" condition have leakages well below the proposed leakage criteria. These low leakage valves have resulted in over 50% of the penetrations being placed in a low leakage category.

The staff has reviewed all licensee submittals concerning the exemption requests and the proposed nine item test program summarized above. The staff finds that the unusual circumstances of the unpredictable timing of the spring snowmelt in the Pacific Northwest and its impact on the refueling schedule for WNP-2 creates an exceptional circumstance for the Supply System that warrants additional considerations relative to the imposition of the strict requirements of Appendix J. The staff also finds that the licensee's

proposal for testing, summarized above as the nine point test program in conjunction with the proposed exemptions, fully meets the intent of Appendix J. Therefore, the licensee's proposed exemptions are acceptable.

Furthermore, based on the testing program proposed by the licensee in the series of seven letters identified above and summarized by the staff as a nine-point testing program, the licensee's proposed technical specification change (January 31, 1986 letter) is acceptable.

IV.

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12, this exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. The Commission further determines that special circumstances, as provided in 10 CFR 50.12(a)(2)(iii), are present justifying the exemption, namely that application of the regulation in the particular circumstances would result in undue hardship and other costs that are significantly in excess of those contemplated when the regulation was adopted and that are significantly in excess of those incurred by others similarly situated. If the plant were forced to undergo Type C testing, solely to comply with the Appendix J regulation, an undue hardship and financial burden would result that would be significantly in excess of that contemplated when the regulation was adopted. When the regulation was adopted, it was contemplated that the testing would be accomplished during the normally anticipated and scheduled refueling outages that occur in most plants approximately every eighteen months. Thus the cost and hardship imposed on WNP-2 by failing to grant the exemption would be considerably in excess of that incurred by others similarly situated. Therefore the Commission hereby approves the following exemption requests:

Type C testing of containment isolation valves, as required by 10 CFR Part 50, Appendix J, Section III.D.3, Type C tests, need not be performed during each reactor shutdown for refueling but may be performed at other convenient intervals. The interval between successive Type B or Type C tests shall not exceed 27 months.

It is further determined that the exemption does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. In light of this determination and as reflected in the Environmental Assessment and Finding of No Significant Impact prepared pursuant to 10 CFR 51.2 and 51.30 through 51.32, it is concluded the instant action is insignificant from the standpoint of environmental impact and an environmental impact statement need not be prepared.

For further details with respect to this action, see the licensee's request dated January 31, 1986, which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Richland Public Library, Swift and Northgate, Richland, Washington 99352. Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this Exemption will have no significant impact on the environment (52 FR 10834 dated April 3, 1987).

This exemption is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by

Gary M. Holahan, Acting Director
Division of Reactor Projects - III/IV/V
& Special Projects

Dated at Bethesda, Maryland
this 29th day of April, 1987.

*See Previous Concurrence

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FOR THE NUCLEAR REGULATORY COMMISSION

Dennis Crutchfield, Director
Division Reactor Projects-III/IV/V
and Special Projects
Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland
this day of , 1986.

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Dennis M. Crutchfield, Director
Division Reactor Projects - III/IV/V
& Special Projects

Dated at Bethesda, Maryland
this day of , 1986.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21
Amendment No. 41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Washington Public Power Supply System, (the Supply System or licensee) dated January 31, 1986, and as clarified by letters dated April 11 and July 22, 1986, and January 9, February 11, March 4, and April 17, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's Rules and Regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulation;
 - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. NPF-21 is amended to revise the Technical Specifications as indicated in the attachment to this amendment, and paragraph 2.C.(2) of Facility Operating License is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 41, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
Gary M. Holahan for

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III/IV/V
& Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: 29 APR 1987

*See Previous Concurrence

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4/28/87

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DCrutchfield
4/29/87

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director
Project Directorate v
Division of Reactor Projects-III/IV/V
and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance:

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ATTACHMENT TO LICENSE AMENDMENT NO.41
FACILITY OPERATING LICENSE NO. NPF-21
DOCKET NO. 50-397

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Remove Page

3/4 6-4

Insert Page

3/4 6-4

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. Type B and C tests shall be conducted with gas at P_a , 34.7 psig,* at intervals no greater than 24*** months except for tests involving:
 - 1. Air Locks
 - 2. Main steam line isolation valves,
 - 3. Valves pressurized with fluid from a seal system,
 - 4. ECCS and RCIC containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 - 5. Purge supply and exhaust isolation valves with resilient seals.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least $1.10 P_a$, 38.2 psig, and the seal system capacity is adequate to maintain system pressure for at least 30 days.
- h. ECCS and RCIC containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at least once per 18 months.
- i. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE per Surveillance Requirements 4.6.1.8.2 and 4.6.1.8.3.
- j. The provisions of Specification 4.0.2 are not applicable to 24-month or 40 ± 10 -month surveillance intervals.

*Unless a hydrostatic test is required per Table 3.6.3-1.

***For those tests conducted during refueling outages, the 24-month interval may be exceeded by no more than 3 months.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. NPF-21
WASHINGTON PUBLIC POWER SUPPLY SYSTEM
WPPSS NUCLEAR PROJECT NO. 2
DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated January 31, 1986, the Washington Public Power Supply System, the licensee for WNP-2, requested exemptions from the requirements of 10 CFR 50, Appendix J, regarding local leak rate test schedules and proposed Technical Specifications (TS) changes to reflect these exemptions. The exemption requests concern the requirements contained in Paragraphs III.D.2(a) and III.D.3 of Appendix J. Specifically, these paragraphs contain requirements that Type C leakage tests be performed on specified containment barriers during each reactor shutdown for refueling, but in no case shall Type B or C testing be done at intervals greater than two years. In lieu of these requirements, the licensee proposed to divide the components requiring Type B or C testing into two roughly equivalent groups to receive less frequent testing. Only one of the two groups would be tested during any given refueling outage, alternating the group to be tested so that all barriers would be tested at least within two refueling cycles. Additionally, the licensee requested a three-month allowance on the maximum interval between tests. The maximum interval would thereby become 27 months, rather than the 24 months required currently. The licensee also indicated that a program would be in place to identify barriers that leak excessively. When so identified, these barriers would be leak tested every refueling outage until the measured "as found" leakage is restored to an acceptable value.

2.0 EVALUATION

The safety evaluation is contained in Section III of the Exemption document which is enclosed.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comments on such finding.

Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register on February 26, 1987 (52 FR 5871) and consulted with the State of Washington. No public comments were received, and the State of Washington did not have any comments.

We have concluded, based on the considerations discussed above and in the Exemption, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Gwo, NRR

Dated: 29 APR 1987