



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 13, 1986

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P.O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

Subject: Issuance of Amendment No. 24 to Facility Operating
License No. NPF-21 - WPPSS Nuclear Project No. 2

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 24 to Facility Operating License No. NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated January 17, 1986.

This amendment revises the WNP-2 Technical Specifications by changing the operation of valve FPC-V-149 from manual to automatic. Valve FPC-V-149 is the interface between the Suppression Pool Cleanup System and the Fuel Pool Cooling (FPC) System. At present it is manually closed and locked during normal power operation. The change will provide a motorized isolation valve in the suppression pool cleanup return line and allow use of the FPC filter-demineralizers for suppression pool cleanup during reactor operations.

A copy of the related safety evaluation supporting Amendment No. 24 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosures:

1. Amendment No. 24 to Facility
Operating License No. NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

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Certified By la

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P PDR

Mr. G. C. Sorensen, Manager
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. NPF-21
WPPSS NUCLEAR PROJECT NO. 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 24
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee), dated January 17, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 24, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Date of Issuance: May 13, 1986

ENCLOSURE TO LICENSE AMENDMENT NO. 24

FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-22
3/4 6-25
3/4 6-26
3/4 8-26

INSERT

3/4 6-22
3/4 6-25
3/4 6-26
3/4 8-26

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
a. <u>Automatic Isolation Valves (Continued)</u>		
Equipment Drain (Radioactive)	4	15
EDR-V-19		
EDR-V-20		
Floor Drain (Radioactive)	4	15
FDR-V-3		
FDR-V-4		
Fuel Pool Cooling/Suppression Pool Cleanup	4	35
FPC-V-149		
FPC-V-153(f)		
FPC-V-154(f)		
FPC-V-156		
Reactor Recirculation Hydraulic Control(e)	4	5
HY-V-17A,B		
HY-V-18A,B		
HY-V-19A,B		
HY-V-20A,B		
HY-V-33A,B		
HY-V-34A,B		
HY-V-35A,B		
HY-V-36A,B		
Traversing Incore Probe Valve	4	5
TIP-V-6,7,8,9,10 (Probe Line Ball Valves)		
TIP-V-11 (N ₂ Gate Valve)		

WASHINGTON NUCLEAR - UNIT 2

3/4 6-22

Amendment No. 24

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
b. <u>Excess Flow Check Valves (e) (Continued)</u>		
Reactor Pressure Vessel		N.A.
PI-EFC-X12A,B,C		
PI-EFC-X18A,B,C,D		
PI-EFC-X37e,f		
PI-EFC-X38a,b,c,d,e,f		
PI-EFC-X39a,b,d,e		
PI-EFC-X40c,d		
PI-EFC-X41c,d		
PI-EFC-X42a,b		
PI-EFC-X44Aa,Ab,Ac,Ad,Ae,Af,Ag,Ah,Aj, Ak,Al,Am		
PI-EFC-X44Ba,Bb,Bc,Bd,Be,Bf,Bg,Bh,Bj, Bk,Bl,Bm		
PI-EFC-X61a,b		
PI-EFC-X62c,d		
PI-EFC-X66		
PI-EFC-X67		
PI-EFC-X69a,b,e		
PI-EFC-X70a,b,c,d,e,f		
PI-EFC-X71a,b,c,d,e,f		
PI-EFC-X72a		
PI-EFC-X73a		
PI-EFC-X74a,b,e,f		
PI-EFC-X75a,b,c,d,e,f		
PI-EFC-X78b,c,f		
PI-EFC-X79a,b		
PI-EFC-X82b		
PI-EFC-X84a		
PI-EFC-X106		
PI-EFC-X107		
PI-EFC-X108		

TABLE 3.6.3-1 (Continued)
PRIMARY CONTAINMENT ISOLATION VALVES

<u>VALVE FUNCTION AND NUMBER</u>	<u>VALVE GROUP(a)</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>
b. <u>Excess Flow Check Valves (e) (Continued)</u>		
Reactor Pressure Vessel (Continued)		N. A.
PI-EFC-X109		
PI-EFC-X110		
PI-EFC-X111		
PI-EFC-X112		
PI-EFC-X113		
PI-EFC-X114		
PI-EFC-X115		
PI-EFC-X119		
Other		N. A.
PI-EFC-X40e, f		
PI-EFC-X41e, f		
PI-EFC-X86A, B		
PI-EFC-X87A, B		
c. <u>Manual Containment Isolation Valves</u>		
Demineralized Water		N. A.
DW-V-156		
DW-V-157		
Containment Air System		N. A.
CAS-VX-82e		
Service Air		N. A.
SA-V-109		

TABLE 3.8.4.3-1

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION

<u>VALVE NUMBER</u>	<u>SYSTEM(S) AFFECTED</u>	<u>SYSTEM(S) VALVE NUMBER</u>	<u>AFFECTED</u>
a. CAC-V-2 CAC-V-4 CAC-V-6 CAC-V-8 CAC-V-11 CAC-V-13 CAC-V-15 CAC-V-17	Containment Atmospheric Control System	g. MSLC-V-1A MSLC-V-1B MSLC-V-1C MSLC-V-1D MSLC-V-2A MSLC-V-2B MSLC-V-2C MSLC-V-2D MSLC-V-3A MSLC-V-3B MSLC-V-3C MSLC-V-3D MSLC-V-4 MSLC-V-5 MSLC-V-9 MSLC-V-10	Main Steam Isolation Valve Leakage Control System
b. CIA-V-20 CIA-V-30A CIA-V-30B	Containment Instrument Air System		
c. FPC-V-149 FPC-V-153 FPC-V-154 FPC-V-156	Fuel Pool Cooling System		
d. HPCS-V-1 HPCS-V-4 HPCS-V-10 HPCS-V-11 HPCS-V-12 HPCS-V-15 HPCS-V-23	High Pressure Core Spray System	h. RCC-V-5 RCC-V-6 RCC-V-17A RCC-V-17B RCC-V-21 RCC-V-40 RCC-V-71A RCC-V-71B RCC-V-71C RCC-V-72A RCC-V-72B RCC-V-104 RCC-V-129 RCC-V-130 RCC-V-131	Reactor Closed Cooling Water System
e. LPCS-V-1 LPCS-V-5 LPCS-FCV-11 LPCS-V-12	Low Pressure Core Spray System		
f. MS-V-1 MS-V-2 MS-V-5 MS-V-16 MS-V-19 MS-V-20 MS-V-67A MS-V-67B MS-V-67C MS-V-67D MS-V-146	Main Steam System	i. RCIC-V-1 RCIC-V-8 RCIC-V-10 RCIC-V-13 RCIC-V-19 RCIC-V-22 RCIC-V-31	Reactor Core Isolation Cooling System



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 24 TO FACILITY OPERATING LICENSE NO. NPF-21

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

WPPSS NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated January 17, 1986, the Supply System requested a number of revisions to the WNP-2 Technical Specifications, Table 3.6.3-1 (Primary Isolation Valves). One of the revisions involved the conversion of a locked-closed, manual isolation valve to a normally closed, motor-operated isolation valve. To permit early implementation of the capability provided by this modification, this safety evaluation addresses only this valve conversion item. The other proposed revisions requested by the Supply System will be addressed in a separate licensing action.

2.0 EVALUATION

The proposed revision involves a valve designated by the licensee as FPC-V-149. At present this valve is listed in section 'c' of Table 3.6.3-1 of the WNP-2 Technical Specifications (page 3/4 6-26) as a Manual Containment Isolation Valve and is described as a "Fuel Pool/Suppression Pool Cooling" valve. Table 6.2-12 of the Final Safety Analysis Report (FSAR) for the facility further describes the valve as a six-inch, manually operated, globe valve that is normally locked closed. Table 6.2-12 (page 6.2-129) also indicates valve FPC-V-149 is one of four Containment Isolation Valves provided for the Suppression Pool Cleanup suction and return lines. The other three isolation valves are FPC-V-153, 154 and 156. Table 6.2-12 indicates these three valves are six-inch, motor-operated gate valves that are normally closed. The Table also indicates these valves receive an isolation signal upon occurrence of low water level in the reactor vessel or high pressure in the drywell.

The revision proposed by the licensee would change the operation of the valve and move FPC-V-149 from Section 'c' (Manual Isolation Valves) of the Technical Specification Table 3.6.3-1 to Section 'a' (Automatic Isolation Valves). The valve would thus be grouped with the other Fuel Pool/Suppression Pool valves listed above. The maximum stroke time for FPC-V-149 stated in Table 3.6.3-1 would be the same as for the other three valves.

This proposed revision is prompted by the licensee's plant modification which has resulted in FPC-V-149 being equipped with a motor operator and provided with an automatic isolation signal. Further the actual modification involved replacing the six-inch, manual globe valve with a six-inch,

motor-operated gate valve. FPC-V-149, therefore, is basically similar to the other automatic isolation valves described above. The licensee states the modification will (1) allow the Fuel Pool Cooling (FPC) system demineralizers also to be used for cleanup of the suppression pool water inventory; and (2) allow the FPC system to be used to maintain water level in the suppression pool. The licensee adds that "Without the ability to use this automatic valve, suppression pool level will have to be maintained utilizing an RHR pump, thereby unnecessarily challenging an ECCS function."

The acceptability of the proposed revision to the Technical Specifications hinges on the acceptability of the substitution of a motor-operated isolation valve for the present locked-closed manual valve (FPC-V-149). Based on Criterion 56, which allows either type of valve to provide the required isolation function, we conclude the modification is appropriate and, accordingly, the proposed revision to the WNP-2 Technical Specification is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation and use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (51 FR 8603) on March 12, 1986, and consulted with the state of Washington. No public comments were received, and the state of Washington did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Gerald B. Zwetzig, Region V

Dated: May 13, 1986