50-387/388

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USER INFORMATION:

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TRANSMITTAL INFORMATION:

~~TO: ELAIM*LAUREL B~~ 07/02/2002
LOCATION: DOCUMENT CONTROL DESK
FROM: NUCLEAR RECORDS DOCUMENT CONTROL CENTER (NUCSA-2)
THE FOLLOWING CHANGES HAVE OCCURRED TO THE HARDCOPY OR ELECTRONIC MANUAL ASSIGNED TO YOU:

126 - 126 - CONTROL ROOM (CR) COMMUNICATOR

REMOVE MANUAL TABLE OF CONTENTS DATE: 05/08/2002

ADD MANUAL TABLE OF CONTENTS DATE: 07/01/2002

CATEGORY: PROCEDURES TYPE: EP
ID: EP-PS-126
ADD: PCAF 2002-1441 REV: N/A

UPDATES FOR HARD COPY MANUALS WILL BE DISTRIBUTED WITHIN 5 DAYS IN ACCORDANCE WITH DEPARTMENT PROCEDURES. PLEASE MAKE ALL CHANGES AND ACKNOWLEDGE COMPLETE IN YOUR NIMS INBOX UPON RECEIPT OF HARD COPY. FOR ELECTRONIC MANUAL USERS, ELECTRONICALLY REVIEW THE APPROPRIATE DOCUMENTS AND ACKNOWLEDGE COMPLETE IN YOUR NIMS INBOX.

A045

PROCEDURE CHANGE PROCESS FORM

1. PCAF NO. 2002-1441	2. PAGE 1 OF 6	3. PROC. NO. EP-PS-126 REV. 17

4. FORMS REVISED - A R20, - F R0, - ___ R___, - ___ R___, - ___ R___, - ___ R___

5. PROCEDURE TITLE
CONTROL ROOM COMMUNICATOR: Emergency-Plan-Position-Specific Instruction

6. REQUESTED CHANGE
PERIODIC REVIEW ☒ NO ☐ YES
INCORPORATE PCAFS ☒ NO ☐ YES # _____ # _____ # _____ # _____
REVISION ☐ PCAF ☒ DELETION ☐ (CHECK ONE ONLY)

7. SUMMARY OF / REASON FOR CHANGE

1) Tab A: Changed "Assembly Area" to "Accountability Area"

2) Tab ~~D~~ F: New tab added to address Security threat or event communications resulting from issuance of NRC order for "Interim Safeguards and Compensatory Measures for SSES".

Continued ☐

8. DETERMINE COMMITTEE REVIEW REQUIREMENTS (Refer to Section 6.1.4)
PORC REVIEW REQ'D? ☒ NO ☐ YES

9. PORC MTG# NA

BLOCKS 11 THRU 16 ARE ON PAGE 2 OF FORM

17. William Tabor / 3665 / 06/04/2002
PREPARER (Print or Type) / ETN / DATE

18. COMMUNICATION OF CHANGE REQUIRED?
☐ NO ☒ YES (TYPE) OPS Buttetin

19. _____ 6/11/02
RESPONSIBLE SUPERVISOR / DATE

SIGNATURE ATTESTS THAT RESPONSIBLE SUPERVISOR HAS CONDUCTED QADR AND TECHNICAL REVIEW UNLESS OTHERWISE DOCUMENTED IN BLOCK 16 OR ATTACHED REVIEW FORMS. CROSS DISCIPLINE REVIEW (IF REQUIRED) HAS BEEN COMPLETED BY SIGNATURE IN BLOCK 16 OR ATTACHED REVIEW FORMS.

20. _____ 6/12/02
FUM APPROVAL / DATE

21. RESPONSIBLE APPROVER — ENTER N/A IF FUM HAS APPROVAL AUTHORITY
_____ 6/13/02
INITIALS / DATE

CONTROLLED

PROCEDURE CHANGE PROCESS FORM

1. PCAF NO. 2002-1441	2. PAGE 2 OF 6	3. PROC. NO. EP-PS-126	REV. 17

11.	This question documents the outcome of the 50.59 and 72.48 Review required by NDAP-QA-0726. Either 11a, b, c or d must be checked "YES" and the appropriate form attached or referenced.		
	a. This change is an Administrative Correction for which 50.59 and 72.48 are not applicable.	☐ YES	☒ N/A
	b. This change is a change to any surveillance, maintenance or administrative procedure for which 50.59 and 72.48 are not applicable..	☒ YES	☐ N/A
	c. This change is bounded by a 50.59/72.48 Screen/Evaluation, therefore, no new 50.59/72.48 Evaluation is required. Screen/Evaluation No. _____	☐ YES	☒ N/A
	d. 50.59 and/or 72.48 are applicable to this change and a 50.59/72.48 Screen/Evaluation is attached.	☐ YES	☒ N/A
12.	This change is consistent with the FSAR or an FSAR change is required. Change Request No. _____	☒ YES	
13.	Should this change be reviewed for potential effects on Training Needs or Material? If YES, enter an Action Item @ NIMS/Action/Gen Work Mech/PICN	☐ YES	☒ NO
14.	Is a Surveillance Procedure Review Checklist required per NDAP-QA-0722?	☐ YES	☒ NO
15.	Is a Special, Infrequent or Complex Test/Evolution Analysis Form required per NDAP-QA-0320? (SICT/E form does not need to be attached.)	☐ YES	☒ NO

16. Reviews may be documented below or by attaching Document Review Forms NDAP-QA-0101-1.

REVIEW	REVIEWED BY WITH NO COMMENTS	DATE
QADR		
TECHNICAL REVIEW		
REACTOR ENGINEERING/NUCLEAR FUELS *		
IST **		
OPERATIONS		
NUCLEAR SYSTEMS ENGINEERING		
NUCLEAR MODIFICATIONS		
MAINTENANCE		
HEALTH PHYSICS		
NUCLEAR TECHNOLOGY		
CHEMISTRY		
OTHER 10 CFR 50.54q	William Dalton	6-4-02

* Required for changes that affect, or have potential for affecting core reactivity, nuclear fuel, core power level indication or impact the thermal power heat balance. (58)

** Required for changes to Section XI Inservice Test Acceptance Criteria.

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PAGE 3 OF 6

CONTROL ROOM COMMUNICATOR: Emergency Plan-Position Specific Procedure

WHEN: Emergency plan is activated

HOW NOTIFIED: Assigned while on duty

REPORT TO: Shift Supervisor/ED

WHERE TO REPORT: Control Room

OVERALL DUTY:

Communicate information on emergency to specified personnel, agencies, and organizations, until the Technical Support Center (TSC) takes over communications.

MAJOR TASKS:	TAB:	REVISION:
Using the "Control Room Communicator's Flowchart," make initial notifications to off-site agencies and emergency personnel.	TAB A	~~21~~ 22 PCAF
Pass on information and documentation when your communications job is finished, due to shift turnover, transfer of control to the TSC, or termination.	TAB B	3
Perform notifications during a hazardous materials spill.	TAB C	3
Perform Security Threat or Event communications as directed by Emergency Director	TAB F	0 PCAF

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SUPPORTING INFORMATION:	TAB:
Emergency Telephone Instructions	TAB 1
Emergency Organization	TAB 2
Logkeeping	TAB 3
Notification Matrix	TAB 4
Emergency Forms • Emergency Notification Report • Emergency Notification Logsheet • Protective Action Recommendation Form	TAB 5
Brief Non-Technical Descriptions of EAL	TAB 6
Emergency Classification	TAB 7

REFERENCES:

NUREG-0654, Planning Standards and Evaluation Criteria

NUREG-0696, Functional Criteria for Emergency Response Facilities

NUREG-0731, Guidelines for Utility Management Structure and Technical Resources, September 1980

SSES Emergency Plan

NDAP-00-0316, Station Communications

NDAP-QA-0300, Conduct of Operations

OP-AD-003, Shift Routine - Log taking

Letter dated February 25, 2002 from Samuel J. Collins, Director, Office of Nuclear Reactor Regulations, to Robert G. Byram, Senior Vice President and Chief Nuclear Officer. | PCAF

EP-PS-126 CONTROL ROOM COMMUNICATOR FLOW CHART

Activation of NERO pagers using the telephone "HOTBUTTON"

- Remove the telephone headset from the twenty button telephone on the center console
- Press the "GREEN" activation button, (the fifth button in the first row of ten)
- After approximately thirty seconds reset the "GREEN" button by pressing any other button and hang up the telephone handset

NOTE:

If the green button is not reset, the pagers will activate again when the handset is picking up

4

Site Evacuation Ordered By ED

E1 Site Accountability Performed

E2 Notify the following that a Site Evacuation of all non-emergency personnel has been ordered: [1] [2]

Dial CTN 191
- ___ PENNA EMA
- ___ Columbia County EMA
- ___ Luzerne County EMA

E3 Document on Emergency Notification Log Sheet

Shift Supervisor Declares Emergency Classification EP-PS-100

Utilize Emergency Notification Log Sheet when entering or changing emergency classification and for logging all actions taken

C1 Make Plant Announcement
Attention all Personnel
- ___ An Unusual Event
- ___ An Alert
- ___ A Site Area Emergency
- ___ A General Emergency

Has been declared at SSES

(Repeat Announcement)

Is it an Unusual Event?

☐ YES

Within fifteen min. of declaration, use CTN "191" to transmit the ENR to: [1] [2] [3]
- ___ Penna. Emergency Management Agency
- ___ Columbia County EMA
- ___ Luzerne County EMA
- ___ Media Operations Center (when staffed)

C2a

Perform Site Accountability, if requested by the Emergency Director [5]

C2b

Activate the NERO, if requested by the Emergency Director [4]

C2c

Contact the ASCC and request notification to key managers [1]

C2d

☐ NO

C2e Activate "NERO" pagers using telephone "HOT BUTTON" [4]

C2f Contact the ASCC: [1]
- ___ Verify pager activation
- ___ Request TNS activation

Within fifteen min. of declaration, use CTN "191" to transmit the ENR to: [1] [2] [3]
- ___ Penna. Emergency Management Agency
- ___ Columbia County EMA
- ___ Luzerne County EMA
- ___ Media Operations Center (when staffed)

C2g

C2h Perform Site Accountability [5]

C3 Call Transmission Power Dispatcher and give emergency classification [1]

C4 Within 60 min. of declaration, transmit Emergency notification Report and Protective Action Recommendation Form, (If release in progress), to the NRC using the ENS telephone [1]

C5 Within two hours of declaration, transmit the emergency classification to off-site agencies: [1]
- ___ Institue of Nuclear Power (INPO)
- ___ Allegheny Electric Cooperative
- ___ American Nuclear Insurers, (ANI)
- ___ PPL Insurance Group

Has Emergency Classification Changed?

☐ NO

C6 Turn over notification to the TSC Communicator(s) when conditions allow

C7 Every hour on the 1/2 hour transmit the Emergency Notification Report providing a static update to: [1] [2] [3]
- ___ Penna. EMA
- ___ Columbia County EMA
- ___ Luzerne County EMA
- ___ Media Operations Center

C8 Every hour on the 1/2 hour, transmit the Emergency Notification Report, (providing a status update), and the Protective Action Recommendation Form, (if a release is in progress), to: [1] [2]
- ___ PIM (CTN 4903)
- ___ DEP/BRP (CTN 4964/4965)
- ___ NRC (ENS line/number on handset)

☐ YES

If upgrading from an Unusual Event:
- ___ Activate NERO pagers using Telephone "Hotbutton" [4]
- ___ Contact ASCC to verify pager activation and request TNS activation [1]
- ___ Perform site accountability, if not previously performed [5]

C9

MAKE PLANT ANNOUNCEMENT

ATTENTION ALL PERSONNEL THE EMERGENCY HAS BEEN UPGRADED/DOWNGRADED TO:
- ___ Unusual Event
- ___ Alert
- ___ Site Area Emergency
- ___ General Emergency

REPEAT ANNOUNCEMENT

C10

Has Emergency Been Terminated?

☐ NO

C11 Within fifteen minutes of classification change, transmit the Emergency Notification Report to: [1] [2] [3]
- ___ Penna. EMA
- ___ Columbia County EMA
- ___ Luzerne County EMA
- ___ Media Operations Center

C12 Notify the following when the emergency has been upgraded/ downgraded
- ___ Emergency Director, TSC
- ___ Recovery Manager

C13 If not previously performed, activate NERO pagers [1] [4]

C14 Call the Transmission Power Dispatcher using the "TPD HOTLINE" and give the Emergency Classification [1] [2]

C15 Transmit the Emergency Notification Report and Protective Action Recommendation form (if release is in progress), to: [1] [2]
- ___ PIM (CTN 4903)

WITHIN ONE HOUR OF CLASSIFICATION CHANGE TO:
- ___ NRC (number(s) on handset)

☐ YES

Notify the following that emergency is terminated: [1] [2] [3]
1. Dial CTN 191:
   - ___ Penna. EMA
   - ___ Columbia County EMA
   - ___ Luzerne County EMA
   - ___ Media Operations Center
2. Refer to On-Call List:
   - ___ Emergency Director, TSC
   - ___ Recovery Manager
3. TPD Hotline:
   - ___ Transmission Power Dispatcher
4. ENS Telephone:
   - ___ NRC (number on handset)
5. INPO
6. Allegheny Electric
7. ANI
8. PPL Insurance Department

C16

Site Accountability ordered By ED or directed by EP-PS-126 Flow Chart

A1 Ensure "Page Merge" pushbutton depressed at Unit 2 US desk

At Panel 0C695
- Turn the siren tone generator switch to "pulsed" position
- Pull out evacuation alarm switch, turn to "Plant Alarm" position and push in Evacuation Alarm switch
- After 30 seconds pull out Evacuation Alarm switch, turn to "Off" position and push in Evacuation Alarm switch

A2

MAKE PLANT ANNOUNCEMENT

"Attention all personnel, a Site Accountability is ordered. All on-call TSC emergency personnel report to your duty stations. All other personnel report to your designated ~~assembly~~ accountability areas."

(Add any additional information regarding habitability as requested by the ED)

(Repeat Announcement)

A3

5 A4 Document on Emergency Notification Log sheet

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Cautions and Notes	
Agency	Emergency Telephone Numbers
PEMA	CTN 4960/4961 (717)651-2148
CCEMA*	CTN 4955/4956 (570)389-5720
LCEMA*	CTN 4906/4907 (800)821-3715
NRC	1-(301)816-5100 1-(301)951-0550
TPD	8-220-5020 8-220-5091
ASCC	CTN 4918 (800)321-0614
INPO	(Within two hours of declaration)
Allegheny Electric Co-operative	(717)233-5704 (Within two hours of declaration)
American Nuclear Insurers	(860)561-3433 (Within two hours of declaration)
PPL Insurance	J. Diacogiannis 220-4961 (W) (610)865-4410 (H) T. Novatnack 220-4962 (W) (610)867-7296 (H) (Within two hours of classification)
DEP/BRP	(717) 787-9135 or (717) 651-2128
Public Information Manager	CTN 4901/4902

* CCEMA & LCEMA can also be contacted via VHF radio located in the TSC.

1

2 Direct request for additional information
From the NRC to the SS/ED
From other agencies to the MOC

3 If MOC does not answer, do not wait, transmit message to emergency agencies

EP-PS-126	SH 1 OF 1
CONTROL ROOM COMMUNICATOR FLOW CHART	
SUSQUEHANNA SES EMERGENCY OPERATING PROCEDURE	
REV. NO. ~~-01-~~	
EP126CFC.CDR	

PCAF 2002-1441

MAJOR TASK;

Perform Site Specific Security Threat of Event emergency communications as directed by the Emergency Director.

SPECIFIC TASK:	HOW:	
1. **Commence** logging communication actions.	1a. **Use** Tab 5 Log Sheet.	____
2. **Transmit** Emergency Notification Report for EAL and Site Evacuation.	2a. **Dial** 191 conference bridge on CTN extension or **USE** Tab 4 backup phone numbers	____
	2b. **Contact:**	
	1) Pennsylvania EMA	____
	2) Columbia County EMA	____
	3) Luzerne County EMA	____
3. **If directed** by the Emergency Director.	3a. **For** evacuation, make a page announcement twice: "Attention all personnel, a security emergency has been declared, all non-essential personnel evacuate the site. All NERO responders report to the Emergency Operations Facility.	____
4. **When** a site accountability is ordered by the Emergency Director.	4a. **Ensure** the "Page Merge" button is depressed.	____
	4b. **Go** to Panel OC695: -	
	1) Turn siren tone generator switch to "pulsed".	____
	2) Pull out evacuation alarm switch.	____
	3) Turn evacuation alarm switch to "plant alarm".	____
	4) Push in evacuation alarm switch	____
	5) Wait 30 seconds.	____
	6) Pull out evacuation alarm switch.	____
	7) Turn evacuation alarm switch to "off".	____
	8) Push in evacuation alarm switch.	
	4c. Make a page announcement twice: "Attention all personnel, a site accountability is ordered. All personnel report to the nearest accountability area."	____
5. **When** Tasks 3 and 4 are completed.	5a. **Enter** Communicator Flowchart step C3.	____

PCAF

BRIEF NON-TECHNICAL DESCRIPTIONS OF EAL

EAL# 1.1:	Aircraft/Train Activity *Unusual Event*

Brief Non-Technical Description:

(Aircraft crash/Train derailment) within the plant security fence (protected area). Based on current plant conditions, the safety of the general public is not threatened.

EAL# 1.2:	Aircraft/Train Activity *Alert*

Brief Non-Technical Description:

(Aircraft crash/Projectile strike) onto a permanent plant structure. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 1.3:	Aircraft/Train Activity *Site Emergency*

Brief Non-Technical Description:

(Aircraft crash/Projectile strike) which caused damage to some equipment used to safely shut down the reactor.

EAL# 2.2:	Control Room Evacuation *Alert*

Brief Non-Technical Description:

The Control Room was evacuated, then control of plant systems was established from another location within the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 2.3:	Control Room Evacuation *Site Emergency*

Brief Non-Technical Description:

Within 15 minutes of evacuating the Control Room, operators have been unable to establish remote stations for controlling plant systems.

EAL# 3.1:	Fuel Cladding Degradation *Unusual Event*

Brief Non-Technical Description:

Minor damage has occurred to the metal tubes that hold uranium fuel pellets. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 3.2:	Fuel Cladding Degradation *Alert*

Brief Non-Technical Description:

There has been significant damage to metal tubes that hold uranium fuel pellets. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 3.3:	Fuel Cladding Degradation *Site Emergency*

Brief Non-Technical Description:

Severe damage has taken place to the metal tubes that hold uranium fuel pellets. Abnormally high radiation levels are present in the water which acts as a reactor coolant or in the containment structure surrounding the reactor.

EAL# 3.4.a:	Fuel Cladding Degradation *General Emergency*

Brief Non-Technical Description:

Severe damage to metal tubes that hold uranium fuel pellets. Higher than normal radiation levels in reactor coolant or containment structure surrounding reactor, with a potential for off-site radioactivity release.

EAL# 3.4.b:	Fuel Cladding Degradation *General Emergency*

Brief Non-Technical Description:

Indication of uranium fuel melting. Potential for off-site radioactivity release.

EAL# 4.1:	General *Unusual Event*

Brief Non-Technical Description:

(____Specific Event____) which potentially threatens the safety of the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 4.2:	General *Alert*

Brief Non-Technical Description:

(___Specific Event___) which actually threatens the safety of the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 4.3:	General *Site Emergency*

Brief Non-Technical Description:

(____Specific Event____) has occurred which indicates an (actual/imminent) loss of important plant safety systems.

EAL# 4.4:	General *General Emergency*

Brief Non-Technical Description:

(___Specific Event___) has occurred which indicates an (actual/imminent) major release of radioactivity.

EAL# 5.1:	Injured/Contaminated Personnel *Unusual Event*

Brief Non-Technical Description:

An injured person(s), contaminated with radioactive material, is or has been moved outside the immediate area of the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 6.1:	In-plant High Radiation *Unusual Event*

Brief Non-Technical Description:

High levels of airborne radioactivity have been detected inside the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 6.2:	In-plant High Radiation *Alert*

Brief Non-Technical Description:

Very high levels of radiation have been detected in the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 7.1:	Loss of AC Power *Unusual Event*

Brief Non-Technical Description:

AC electrical power from either off-site or on-site sources is needed to operate plant safety equipment. Either the off-site or on-site source has been lost; however, power is still available from the other source. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 7.2:	Loss of AC Power *Alert*

Brief Non-Technical Description:

All AC electrical power (from both on-site and off-site) needed to operate plant safety equipment has been temporarily lost. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 7.3:	Loss of AC Power *Site Emergency*

Brief Non-Technical Description:

All AC electrical power (from both on-site and off-site) needed to operate plant safety equipment has been lost for a sustained period of time (15 minutes). Based on current plant conditions, the safety of the general public is not threatened.

EAL# 8.2:	Loss of Control Room Alarms and Annunciators *Alert*

Brief Non-Technical Description:

All control room alarms have been lost. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 8.3:	Loss of Control Room Alarms and Annunciators *Site Emergency*

Brief Non-Technical Description:

All control room alarms have been lost in combination with another plant operating problem (plant transient).

EAL# 9.2:	Loss of DC Power *Alert*

Brief Non-Technical Description:

All DC electrical power in the plant has been lost temporarily. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 9.3:	Loss of DC Power *Site Emergency*

Brief Non-Technical Description:

All DC electrical power--needed to operate some plant safety equipment--has been lost for more than 15 minutes.

EAL# 10.2: Loss of Decay Heat Removal Capability
Alert

Brief Non-Technical Description:

While the reactor is shutdown (cold shutdown), equipment needed to maintain reactor water temperature below 200° F has been lost. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 10.3: Loss of Decay Heat Removal Capability
Site Emergency

Brief Non-Technical Description:

While the unit is shutdown, operators are unable to effectively cool the reactor.

EAL# 10.4: Loss of Decay Heat Removal Capability
General Emergency

Brief Non-Technical Description:

Operators are unable to cool the reactor; a release of radioactivity is possible.

EAL# 11.1: Loss of Reactivity Control
Unusual Event

Brief Non-Technical Description:

Reactor power has increased in a way that was not anticipated. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 11.2: Loss of Reactivity Control
Alert

Brief Non-Technical Description:

During attempted reactor shutdown, the reactor's control rods failed to insert fully (scram). Based on current plant conditions, the safety of the general public is not threatened.

EAL# 11.3: Loss of Reactivity Control
Site Emergency

Brief Non-Technical Description:

Operators are unable to shut down and cool the reactor. Reactor control rods failed to fully insert. A back-up chemical control system also failed.

EAL# 11.4: Loss of Reactivity Control
General Emergency

Brief Non-Technical Description:

Operators are unable to shut down and cool down the reactor. Reactor control rods failed to fully insert. The back-up chemical control system also failed. The situation could lead to a radioactivity release.

EAL# 12.1: Loss of Reactor Vessel Inventory
Unusual Event

Brief Non-Technical Description:

To maintain reactor water level, an emergency cooling system has been activated. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 12.2: Loss of Reactor Vessel Inventory
Alert

Brief Non-Technical Description:

Excessive water is leaking from the reactor coolant systems into the containment structure surrounding the reactor vessel. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 12.3: Loss of Reactor Vessel Inventory
Site Emergency

Brief Non-Technical Description:

The ability to maintain water level above the fuel has been lost.

EAL# 12.4.a:	Loss of Reactor Vessel Inventory *General Emergency*

Brief Non-Technical Description:

The ability to maintain an adequate water level in the reactor vessel has been lost; severe fuel damage and release of radioactivity are possible.

EAL# 12.4.b:	Loss of Reactor Vessel Inventory *General Emergency*

Brief Non-Technical Description:

Fuel damage and a reactor coolant leak have occurred, with a potential loss of the ability to contain radioactive releases.

EAL# 13.1: Natural Phenomena
Unusual Event

Brief Non-Technical Description:

(Tornado/ Hurricane/ Earthquake) has struck the plant site. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 13.2: Natural Phenomena
Alert

Brief Non-Technical Description:

(Tornado/ Hurricane/ Earthquake) has struck the plant and could affect plant safety. Based on current plant conditions, the safety of the general public is not threatened. The event may be severe enough to impact plant equipment.

EAL# 13.3: Natural Phenomena
Site Emergency

Brief Non-Technical Description:

Severe (Tornado/ Hurricane/ Earthquake) is affecting plant safety while plant is not in cold shutdown.

EAL# 14.1: On-site Fire/Explosion
Unusual Event

Brief Non-Technical Description:

A (fire/ explosion) has occurred on-site, within the (plant/ security fence). Based on current plant conditions, the safety of the general public is not threatened.

EAL# 14.2: On-Site Fire/Explosion
Alert

Brief Non-Technical Description:

A (fire/ explosion) has occurred on-site that has affected plant operation. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 14.3: On-site Fire/Explosion
Site Emergency

Brief Non-Technical Description:

A (fire/ explosion) has damaged equipment needed to safely shut down the reactor.

EAL# 15.1: Radiological Effluent
Unusual Event

Brief Non-Technical Description:

Radioactivity is being released from the plant that exceeds plant operating license limits (Technical Specifications) for liquid releases or exceeds 2 times plant operating license limits for gaseous releases. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 15.2: Radiological Effluent
Alert

Brief Non-Technical Description:

Radioactivity is being released at levels at least ten times higher than those allowed by the plant operating license limits (Technical Specifications) for liquid release or exceeds 200 times plant operating license limits for gaseous limits. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 15.3: Radiological Effluent
Site Emergency

Brief Non-Technical Description:

Radioactivity is being or has been released which may result in low levels of exposure to people outside the immediate plant area (emergency planning boundary).

EAL# 15.4: Radiological Effluent
General Emergency

Brief Non-Technical Description:

Radioactivity is being released that exceeds federal guidelines which specify actions like sheltering or evacuation to protect the public.

EAL# 16.1: Security Event *Unusual Event*

Brief Non-Technical Description:

An attempt has been made to breach station security or a site-specific credible threat has been received. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 16.2: Security Event *Alert*

Brief Non-Technical Description:

A compromise of Station security has occurred or is imminent. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 16.3: Security Event *Site Emergency*

Brief Non-Technical Description:

Security has been compromised in a way that threatens plant safety systems.

EAL# 16.4: Security Event *General Emergency*

Brief Non-Technical Description:

Security has been compromised in a way that has caused loss of control of some or all vital areas of the plant.

EAL# 17.1: Spent Fuel Related Incident
Unusual Event

Brief Non-Technical Description:

Used fuel assemblies (groups of the metal rods containing irradiated uranium fuel pellets), being stored in the unit's spent fuel pool, are leaking radioactive material. This is causing abnormally high radiation levels in some areas of the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 17.2: Spent Fuel Related Incident
Alert

Brief Non-Technical Description:

Used fuel assemblies (groups of the metal rods containing irradiated uranium fuel pellets), being stored in the unit's spent fuel pool are leaking radioactive material. This is causing very high radiation levels in some areas of the plant. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 17.3.a: Spent Fuel Related Incident
Site Emergency

Brief Non-Technical Description:

Severe damage has occurred to some used fuel assemblies (metal rods containing irradiated uranium fuel pellets) stored in the unit's spent fuel pool. A radioactivity release is possible.

EAL# 17.3.b: Spent Fuel Related Incident
Site Emergency

Brief Non-Technical Description:

Used (irradiated) fuel assemblies in the unit's spent fuel pool are damaged and no longer covered with cooling water. A radioactive release is possible.

EAL# 18.2: Steam Line Break *Alert*

Brief Non-Technical Description:

Isolation valves have failed to completely shut off the flow of radioactive steam from the reactor to the turbine-generator. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 18.3: Steam Line Break *Site Emergency*

Brief Non-Technical Description:

Isolation valves have failed to properly shut and there is significant flow of radioactive steam from the reactor to areas outside the primary containment.

EAL# 19.1: Toxic/Flammable Gases
Unusual Event

Brief Non-Technical Description:

(Toxic/ flammable) gas has been released (near-site/ on-site). Based on current plant conditions, the safety of the general public is not threatened.

EAL# 19.2: Toxic/Flammable Gases
Alert

Brief Non-Technical Description:

(Toxic/ flammable) gas has entered plant facilities. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 19.3: Toxic/Flammable Gases
Site Emergency

Brief Non-Technical Description:

(Toxic/ flammable) gas has entered areas within plant buildings which contain vital safety equipment (plant vital areas).

EAL# 20.1:	Technical Specification Safety Limit *Unusual Event*

Brief Non-Technical Description:

An abnormal plant condition has occurred. Based on current plant conditions, the safety of the general public is not threatened.

EAL# 21.1.a: Irradiated Spent Fuel in Dry Storage
Unusual Event

Brief Non-Technical Description:

While transporting the used (spent) fuel assemblies (groups of rods containing irradiated uranium fuel pellets) from the spent fuel pool to the onsite storage facility, radiological readings indicate that the spent fuel or its container may be damaged.

EAL# 21.1.b: Irradiated Spent Fuel in Dry Storage
Unusual Event

Brief Non-Technical Description:

After the used (spent) fuel assemblies (groups of rods containing irradiated uranium fuel pellets) have been transported from the spent fuel pool and placed in the horizontal storage module (a concrete structure which will house the steel canister of spent fuel), radiological readings indicate that the fuel storage system may be damaged.

EMERGENCY CLASSIFICATION

CHECK ☑

1.0 TIMING OF CLASSIFICATION

☐ 1.1 **UNUSUAL EVENT**

An **UNUSUAL EVENT** shall be declared within 15 minutes of having information necessary to make a declaration.

☐ 1.2 **ALERT**

An **ALERT** shall be declared within 15 minutes of having information necessary to make a declaration.

☐ 1.3 **SITE AREA EMERGENCY**

A **SITE AREA EMERGENCY** shall be declared within 15 minutes of having information necessary to make a declaration.

☐ 1.4 **GENERAL EMERGENCY**

A **GENERAL EMERGENCY** shall be declared within 15 minutes of having information necessary to make a declaration.

CLASSIFICATION OF EMERGENCY CONDITIONS

USE OF EMERGENCY CLASSIFICATION MATRIX

NOTE: CONFIRM THAT INDICATORS AND/OR ALARMS REFLECT ACTUAL CONDITIONS PRIOR TO TAKING ACTION BASED ON THE INDICATOR OR ALARM.

The matrix is worded in a manner that assumes parameter values indicated are the actual conditions present in the plant.

The matrix is designed to make it possible to precisely classify an abnormal occurrence into the proper emergency classification based on detailed Emergency Action Level (EAL) descriptions. It is impossible to anticipate every abnormal occurrence. Therefore, before classifying any abnormal occurrence based on the EALs in the matrix, one should verify that the general conditions prevalent in-plant and offsite meet the general class description of the emergency classification. In addition, prior to classification, one should be aware of the ramifications in-plant and particularly offsite of that classification. Special consideration of offsite consequences should be made prior to declaring a **GENERAL EMERGENCY**.

POLICY STATEMENT ON EMERGENCY DECLARATIONS

Policy

When an emergency action level (EAL) is exceeded but the plant quickly returns below the EAL, the emergency should be declared. If appropriate, the emergency can also be downgraded or terminated at essentially the same time and the Emergency Notification Form can say the emergency was declared and terminated (or downgraded) minutes later.

Example

The main steam lines isolate on high radiation due to a large oil intrusion into the vessel. The isolation on high radiation is a trigger to declare an Alert. If it can quickly be determined that the cause was not damage to the fuel and we should not be in an Alert, then the Alert should be declared and terminated with the same notification report.

Basis

Declaring the emergency will notify interested offsite personnel and will demonstrate that we recognize that we exceeded an EAL. Immediately terminating or downgrading will avoid activation of company and offsite facilities that are not required for this event.

CLASS DESCRIPTIONS

UNUSUAL EVENT - Events that are occurring or have occurred which indicate a potential degradation of the level of safety of the plant. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

ALERT - Events that are occurring or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

SITE AREA EMERGENCY - Events that are occurring or have occurred which involve actual or imminent major failures of plant functions needed for protection of the public. Any releases are not expected to exceed EPA Protective Action Guideline exposure levels except inside the emergency planning boundary.

GENERAL EMERGENCY - Events that are occurring or have occurred which involve actual or imminent substantial core degradation or melting with potential for loss of containment integrity. Expectation is that releases will exceed EPA Protective Action Guideline exposure levels beyond the emergency planning boundary.

CATEGORY INDEX TO THE MATRIX FOR THE CLASSIFICATION OF EMERGENCY CONDITIONS TABLE OF CONTENTS

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1 - AIRCRAFT/TRAIN ACTIVITY

UNUSUAL EVENT

EAL# 1.1 Aircraft crash or train derailment onsite as indicated by:

Visual observation or notification received by control room operator.

ALERT

EAL# 1.2 Aircraft or missile strikes a station structure as indicated by:

Direct observation or notification received by control room operator.

SITE AREA EMERGENCY

EAL# 1.3 Severe damage to safe shutdown equipment from aircraft crash or missile impact when not in cold shutdown, determined by:

(A *and* B *and* C)
A. Direct observation or notification received by control room operator.
and
B. Shift Supervisor evaluation.
and
C. Reactor Coolant temperature greater than 200°F as indicated on Panel 1C651 (2C651).

GENERAL EMERGENCY

EAL# 1.4 None.

2 - CONTROL ROOM EVACUATION

UNUSUAL EVENT

EAL# 2.1 None.

ALERT

EAL# 2.2 Control Room evacuation as indicated by:

(A *and* B)
A. Initiation of control room evacuation procedures.
and
B. Establishment of control of shutdown systems from local stations.

SITE AREA EMERGENCY

EAL# 2.3 Delayed Control Room Evacuation as indicated by:

(A *and* B)
A. Initiation of control room evacuation procedures.
and
B. Shutdown systems control at local stations not established within 15 minutes.

GENERAL EMERGENCY

EAL# 2.4 None.

3 - FUEL CLADDING DEGRADATION

UNUSUAL EVENT

EAL# 3.1 Core degradation as indicated by:

(A *or* B)

A. Valid Off-gas Pre-treatment Monitor high radiation alarm annunciation on Panel 1C651 (2C651) or indication on Panel 1C600 (2C600).

or

B. Reactor coolant activity, determined by sample analysis greater than or equal to 2 μCi/cc of I-131 equivalent.

ALERT

EAL# 3.2 Severe fuel cladding degradation as indicated by:

(A *or* B *or* C *or* D)

A. Valid Off-gas Pre-treatment monitor High-High radiation alarm annunciation on Panel 1C651 (2C651) or indication on Panel 1C600 (2C600).

or

B. Valid Reactor coolant activity greater than 300 μCi/cc of equivalent I-131, as determined by sample analysis.

or

C. Valid Main Steam Line High radiation trip annunciation or indication on Panel 1C651 (2C651).

or

D. Valid containment post accident monitor indication on Panel 1C601 (2C601) greater than 200 R/hr. (An 8R/hr correction factor must be added manually to the indication to offset a downscale error if primary containment temperature exceeds 225 degrees Fahrenheit. Reference EC-079-0521.)

(CONTINUED ON NEXT PAGE)

3 - FUEL CLADDING DEGRADATION (continued)

SITE AREA EMERGENCY

EAL# 3.3 Severely degraded core as indicated by:

(A *or* B)

A. Reactor coolant activity greater than 1,000 μCi/cc of equivalent I-131 as determined by sample analysis.

or

B. Valid containment post accident monitor indication on Panel 1C601 (2C601) greater than 400 R/hr. (An 8 R/hr correction factor must be added manually to the indication to offset a downscale error if primary containment temperature exceeds 225 degrees Fahrenheit. Reference EC-079-0521.)

(CONTINUED ON NEXT PAGE)

3 - FUEL CLADDING DEGRADATION (continued)

GENERAL EMERGENCY

EAL# 3.4.a Fuel cladding degradation. Loss of 2 out of 3 fission product barriers (fuel cladding and reactor coolant pressure boundary) with potential loss of the third barrier (primary containment) as indicated by:

(A *or* B)

A. (1 *and* 2)
   1. Valid containment post accident monitor indication on Panel 1C601 (2C601) greater than 400 R/hr. (An 8 R/hr correction factor must be added manually to the indication to offset a downscale error if primary containment temperature exceeds 225 degrees Fahrenheit. Reference EC-079-0521.)

   and

   2. (a *or* b *or* c)
      a. Containment pressure greater than 40.4 PSIG, indicated on Panel 1C601 (2C601).

      or

      b. A visual inspection of the containment indicates a potential for loss of containment (e.g. anchorage or penetration failure, a crack in containment concrete at tendon).

      or

      c. Other indications of potential or actual loss of primary containment.

or

B. (1 *and* 2)
   1. Reactor coolant activity greater than 1,000 μCi/cc of equivalent I-131 as determined by sample analysis.

   and

   2. Actual or potential failure of reactor coolant isolation valves to isolate a coolant leak outside containment as determined by valve position indication on Panel 1C601 (2C601) or visual inspection.

OR

EAL# 3.4.b Core melt as indicated by:

(A *and* B)

A. Valid containment post accident monitor indication on Panel 1C601 (2C601) greater than 2000 R/hr. (An 8 R/hr correction factor must be added manually to the indication to offset a downscale error if primary containment temperature exceeds 225 degrees Fahrenheit. Reference EC-079-0521.)

and

B. Containment high pressure indication or annunciation on Panel 1C601 (2C601).

4 - GENERAL

UNUSUAL EVENT

EAL# 4.1 Plant conditions exist that warrant increased awareness on the part of plant operating staff or state and/or local offsite authorities as indicated by:

Events that are occurring or have occurred which indicate a potential degradation of the level of safety of the plant. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.

ALERT

EAL# 4.2 Other plant conditions exist that warrant precautionary activation of PP&L, State, County, and local emergency centers as indicated by:

Events that are occurring or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.

SITE AREA EMERGENCY

EAL# 4.3 Other plant conditions exist that warrant activation of emergency centers and monitoring teams or a precautionary notification to the public near the site as indicated by:

Events that are occurring or have occurred which involve actual or imminent major failures of plant functions needed for protection of the public. Any releases are not expected to exceed EPA Protective Action Guideline exposure levels except inside the emergency planning boundary.

GENERAL EMERGENCY

EAL# 4.4 Other plant conditions exist, from whatever, source, that make release of large amounts of radioactivity in a short time period available as indicated by:

Events that are occurring or have occurred which involve actual or imminent substantial core degradation or melting with potential for loss of containment integrity. Expectation is that releases will exceed EPA Protective Action Guideline exposure levels beyond the emergency planning boundary.

5.- INJURED/CONTAMINATED PERSONNEL

UNUSUAL EVENT

EAL# 5.1 Transportation of externally contaminated injured individual from site to offsite medical facility as deemed appropriate by Shift Supervisor.

ALERT

EAL# 5.2 None.

SITE AREA EMERGENCY

EAL# 5.3 None.

GENERAL EMERGENCY

EAL# 5.4 None.

6 - IN-PLANT HIGH RADIATION

UNUSUAL EVENT

EAL# 6.1 Unanticipated or unplanned concentrations of airborne activity exist in normally accessible areas, which are not due to planned maintenance activities, as indicated by:

Concentrations exceed 500 times the DAC values of 10CFR20 Appendix B, Table I values for a single isotope, or for multiple isotopes where

$$\frac{C_A}{DAC_A}+\frac{C_B}{DAC_B}+\frac{C_C}{DAC_C}\cdots\frac{C_N}{DAC_N}\geq 500$$

ALERT

EAL# 6.2 Unexpected in-plant high radiation levels or airborne contamination which indicates a severe degradation in the control of radioactive material as indicated by:

Area Radiation Monitor reading 1000 times normal annunciation on Panel 1C601 (2C601) or indication on Panel 1C600 (2C600).

SITE AREA EMERGENCY

EAL# 6.3 None.

GENERAL EMERGENCY

EAL# 6.4 None.

7 - LOSS OF AC POWER

UNUSUAL EVENT

EAL# 7.1 Loss of offsite power **or** loss of all onsite AC power supplies as indicated by:

(A *or* B)

A. Loss of power to Startup Transformer 10 **and** 20 annunciation or indication on Panel 0C653.

or

B. Failure of all diesel generators to start or synchronize to the emergency buses by indication or annunciation on Panel 0C653.

ALERT

EAL# 7.2 Loss of all offsite power **and** all onsite AC power supplies as indicated by:

(A *and* B)

A. Loss of power to Startup Transformer 10 **and** 20 annunciation or indication on Panel 0C653.

and

B. Failure of all diesel generators to start or synchronize to the emergency buses by annunciation or indication on Panel 0C653.

SITE AREA EMERGENCY

EAL# 7.3 Loss of all offsite power and loss of all onsite AC power supplies for greater than 15 minutes as indicated by:

(A *and* B *and* C)

A. Loss of offsite power.

and

B. Failure of **all** diesel generators to startup or synchronize to the emergency buses by indication or annunciation on 0C653.

and

C. The above conditions exist for greater than 15 minutes.

GENERAL EMERGENCY

EAL# 7.4 None.

8 - LOSS OF CONTROL ROOM ALARMS AND ANNUNCIATORS

UNUSUAL EVENT

EAL# 8.1 None.

ALERT

EAL# 8.2 Loss of all control room annunciators as indicated by:

In the opinion of the Shift Supervisor, all Control Room annunciators and the Plant Process Computer are lost, or insufficient annunciators are available to safely operate the unit(s) without supplemental observation of plant systems.

SITE AREA EMERGENCY

EAL# 8.3 All annunciators lost and plant transient initiated while annunciators are lost as indicated by:

(A *and* B)

A. In the opinion of the Shift Supervisor, all Control Room annunciators and the Plant Process Computer are lost, or insufficient annunciators are available to safely operate the unit(s) without supplemental observation of plant systems.

and

B. (1 *or* 2 *or* 3 *or* 4)

1. Low-Low reactor water level indication on Panel 1C651 (2C651) followed by ECCS initiation on Panel 1C601 (2C601).

or

2. Reactor coolant temperature change greater than 100°F per hour indication on recorder TR-1R006 on Panel 1C007 (2C007) (Reactor Building elevation 683').

or

3. High reactor pressure indication on Panel 1C651 (2C651) and followed by scram indication on Panel 1C651 (2C651).

or

4. Any indication that transient has occurred or is in progress.

GENERAL EMERGENCY

EAL# 8.4 None.

9 - LOSS OF DC POWER

UNUSUAL EVENT

EAL# 9.1 None.

ALERT

EAL# 9.2 Loss of onsite vital DC power as indicated by:

(A *and* B)

A. Less than 210 volts on the 250 VDC main distribution Panel buses, 1D652 (2D652) **and** 1D662 (2D662) as indicated by trouble alarms on Panel 1C651 (2C651).

and

B. Less than 105 volts on the 125 VDC main distribution buses 1D612 (2D612), 1D622 (2D622), 1D632 (2D632), **and** 1D642 (2D642) as indicated by trouble alarms on Panel 1C651 (2C651).

NOTE: Buses are not tripped on undervoltage condition.

SITE AREA EMERGENCY

EAL# 9.3 Loss of all vital onsite DC power sustained for greater than 15 minutes as indicated by:

(A *and* B *and* C)

A. Less than 210 volts on the 250 VDC main distribution Panel buses, 1D652 (2D652) **and** 1D662 (2D662) as indicated by trouble alarms on Panel 1C651 (2C651).

and

B. Less than 105 volts on the 125 VDC main distribution buses 1D612 (2D612), 1D622 (2D622), 1D632 (2D632), **and** 1D642 (2D642) as indicated by trouble alarms on Panel 1C651 (2C651).

and

C. The above condition exists for greater than 15 minutes.

NOTE: Buses are not tripped on undervoltage condition.

GENERAL EMERGENCY

EAL# 9.4 None.

10 - LOSS OF DECAY HEAT REMOVAL CAPABILITY

UNUSUAL EVENT

EAL# 10.1 None.

ALERT

EAL# 10.2 Inability to remove decay heat while in plant condition 4, inability to maintain the plant in cold shutdown as indicated by:

Inability to maintain reactor coolant temperature less than 200°F with the reactor mode switch in shutdown; exception is when testing per Special Test Exception TS 3.10.1 which allows maximum temperature of 212°F.

SITE AREA EMERGENCY

EAL# 10.3 Inability to remove decay heat while the plant is shutdown as indicated by:

(A *and* B *and* C)
A. Reactor Mode switch in shutdown.
and
B. Reactor Coolant System temperature greater than 200°F and rising.
and
C. Suppression Pool temperature greater than 120°F and rising.

GENERAL EMERGENCY

EAL# 10.4 Inability to remove decay heat while the plant is shutdown with possible release of large amounts of radioactivity as indicated by:

(A *and* B *and* C)
A. Reactor mode switch in shutdown.
and
B. Reactor coolant system temperature greater than 200°F and rising.
and
C. Suppression pool temperature greater than 290°F indicated on the computer output (MAT 12,13,14,15 or 16).

11 - LOSS OF REACTIVITY CONTROL

UNUSUAL EVENT

EAL# 11.1 Inadvertent Criticality as indicated by:

Unexpected increasing neutron flux indication on Panel 1C651 (2C651).

ALERT

EAL# 11.2 Failure of the Reactor Protection System or the Alternate Rod Insertion System to initiate and complete a scram that brings the reactor subcritical as indicated by:

(A *or* B) *and* (C *and* D *and* E)

A. Trip of at least one sub-channel in each trip system (RPS A and RPS B) as indicated by annunciators and trip status lights on Panel 1C651 (2C651).

or

B. Trip of both trip systems (ARI A and ARI B) as indicated by annunciators on Panel 1C601 (2C601).

and

C. Failure of control rods to insert, confirmed by the full core display indication on Panel 1C651 (2C651) or process computer indications.

and

D. Failure to bring the reactor subcritical confirmed by neutron count rate on the neutron monitoring indication on Panel 1C651 (2C651).

and

E. Reactor power >5% as indicated on Panel 1C651 (2C651).

(CONTINUED ON NEXT PAGE)

11 - LOSS OF REACTIVITY CONTROL (continued)

SITE AREA EMERGENCY

EAL# 11.3 Loss of functions needed to bring the reactor subcritical and loss of ability to bring the reactor to cold shutdown as indicated by:

(A *and* B *and* C *and* D)

A. Inability to insert sufficient control rods to bring the reactor subcritical as indicated by count rate on the neutron monitoring instrumentation on Panel 1C651 (2C651).

and

B. (1 *or* 2)
Failure of both loops of standby liquid control to inject into the vessel indicated by:
1. Low pump discharge pressure indication on Panel 1C601 (2C601).

or

2. Low flow indication on Panel 1C601 (2C601).

and

C. Reactor coolant temperature greater than 200°F, indicated on Panel 1C651 (2C651).

and

D. Reactor power >5% indicated on Panel 1C651 (2C651).

GENERAL EMERGENCY

EAL# 11.4 Loss of functions needed to bring the reactor subcritical and transient in progress that makes release of large amounts of radioactivity in a short period possible as indicated by:

(A *or* B) *and* (C *and* D)

A. Trip of at least one sub-channel in each trip system (RPS A and RPS B), indicated by annunciation or trip status lights on Panel 1C651 (2C651).

or

B. Trip of both systems (ARI A and ARI B) as indicated by annunciators on Panel 1C601 (2C601).

and

C. Loss of SLC system capability to inject, indicated by instrumentation on Panel 1C601 (2C601).

and

D. Reactor power greater than 25% of rated, indicated on Panel 1C651 (2C651).

12 - LOSS OF REACTOR VESSEL INVENTORY

UNUSUAL EVENT

EAL# 12.1 Valid initiation of an Emergency Core Cooling System (ECCS) System as indicated by:

(A *or* B)

A. Initiation of an ECCS System **and** low, low, low reactor water level (-129) annunciation or indication on Panel 1C651 (2C651).

or

B. Initiation of an ECCS System **and** High Drywell Pressure annunciation or indication on Panel 1C601 (2C601).

ALERT

EAL# 12.2 Reactor coolant system leak rate greater than 50 gpm as indicated by:

(A *or* B)

A. Drywell floor drain sump A or B Hi-Hi alarm on Panel 1C601 (2C601) **and** 2 or more drywell floor drain pumps continuously running as indicated on Panel 1C601 (2C601).

or

B. Other estimates of Reactor coolant system leakage indicating greater than 50 gpm.

SITE AREA EMERGENCY

EAL# 12.3 Known loss of coolant accident greater than make-up capacity as indicated by:

Water level below (and failure to return to) top of active fuel for greater than three minutes as indicated on fuel zone level indicator on Panel 1C601 (2C601).

(CONTINUED ON NEXT PAGE)

12 - LOSS OF REACTOR VESSEL INVENTORY (continued)

GENERAL EMERGENCY

EAL# 12.4.a Loss of coolant accident with possibility of imminent release of large amounts of radioactivity as indicated by:

Water level below (and failure to return to) top of active fuel for greater than 20 minutes as indicated on fuel zone level indicator on Panel 1C601 (2C601).

OR

EAL# 12.4.b Loss of Reactor Vessel inventory. Loss of 2 out of 3 fission product barriers (fuel cladding & reactor coolant pressure boundary) with potential loss of the third barrier (primary containment), as indicated by:

(A *or* B)

A. (1 *and* 2 *and* 3)
   1. High drywell pressure annunciation or indication on Panel 1C601 (2C601).

   and

   2. (a *or* b *or* c)
      a. Containment pressure exceeds 40.4 PSIG as indicated on Panel 1C601 (2C601).

      or

      b. A visual inspection of the containment indicates a potential or actual loss of containment (e.g. anchorage or penetration failure).

      or

      c. Containment isolation valve(s) fail to close as indicated by valve position indication on Panel 1C601 (2C601).

   and

   3. Reactor Vessel level drops below (and fails to return to) top of active fuel for greater than three minutes as indicated on fuel zone level indicator on Panel 1C601 (2C601).

or

B. (1 *and* 2)
   1. Failure of reactor pressure vessel isolation valves to isolate coolant break outside containment as indicated by valve position indication on Panel 1C601 (2C601) or visual inspection.

   and

   2. Reactor vessel level drops below (and fails to return to) top of active fuel for greater than three minutes as indicated on fuel zone level indicator on Panel 1C601 (2C601).

13 - NATURAL PHENOMENA

UNUSUAL EVENT

EAL# 13.1 Natural phenomenon occurrence as indicated by:

(A *or* B *or* C)
A. Tornado impact on site.
or
B. Hurricane impact on site.
or
C. Earthquake detected by seismic instrumentation systems on Panel 0C696.

ALERT

EAL# 13.2 Natural Phenomenon Occurrence as indicated by:

(A *or* B *or* C)
A. Tornado with reported wind velocities greater than 200 mph impacting on site.*
or
B. Reported hurricane or sustained winds greater than 70 mph.*
or
C. Earthquake at greater than operating basis earthquake (OBE) levels as indicated on Panel 0C696.

* **Telephone numbers for the National Weather Bureau are located in the Emergency Telephone Directory.**

(CONTINUED ON NEXT PAGE)

13 - NATURAL PHENOMENA (continued)

SITE AREA EMERGENCY

EAL# 13.3 Severe natural phenomenon occurrence, with plant not in cold shutdown, as indicated by:

(A *and* B)

A. Reactor Coolant Temperature greater than 200°F as indicated on Panel 1C651 (2C651).

and

B. (1 *or* 2 *or* 3)

1. Reported hurricane or sustained winds greater than 80 mph.*

or

2. Earthquake with greater than Safe Shutdown Earthquake (SSE) levels as indicated on Panel 0C696.

or

3. Tornado with reported wind velocities greater than 220 mph impacting on site.*

GENERAL EMERGENCY

EAL# 13.4 None.

* **Telephone numbers for the National Weather Bureau are located in the Emergency Telephone Directory.**

14 - ONSITE FIRE/EXPLOSION

UNUSUAL EVENT

EAL# 14.1 Significant fire within the plant as indicated by:

(A *and* B)
A. Activation of fire brigade by Shift Supervisor.
and
B. Duration of fire longer than 15 minutes after time of notification.

OR

Explosion inside security protected area, with no significant damage to station facilities, as indicated by:

Visual observation or notification received by control room operator and Shift Supervisor evaluation.

ALERT

EAL# 14.2 On-site Fire/Explosion as indicated by:

(A *or* B)
A. Fire lasting more than 15 minutes and fire is in the vicinity of equipment required for safe shutdown of the plant and the fire is damaging or is threatening to damage the equipment due to heat, smoke, flame, or other hazard.
or
B. (1 *and* 2)
Explosion damage to facility affecting plant operation as determined by:
1. Direct observation or notification received by control room operator.
and
2. Shift Supervisor observation.

(CONTINUED ON NEXT PAGE)

14 - ONSITE FIRE/EXPLOSION (continued)

SITE AREA EMERGENCY

EAL# 14.3 Damage to safe shutdown equipment due to fire or explosion has occurred when plant is not in cold shutdown, and damage is causing or threatens malfunction of equipment required for safe shutdown of the plant as determined by:

(A *and* B *and* C)
A. Direct observation or notification received by control room operator.
and
B. Shift Supervisor evaluation.
and
C. Reactor Coolant Temperature greater than 200°F as indicated on Panel 1C651 (2C651).

GENERAL EMERGENCY

EAL# 14.4 None.

15 - RADIOLOGICAL EFFLUENT

UNUSUAL EVENT

EAL# 15.1.a Radiological gaseous effluents exceed 2 times the Technical Requirement Limits for 60 minutes or longer as indicated by:

(A *or* B)

A. Valid Building Vent Stack Monitoring System (SPING) indications on Panel 0C630 or 0C677.
   1. Noble gases >1.70E+6 μCi/min., **or**
   2. I-131 >2.08E+2 μCi/min., **or**
   3. Particulate >1.54E+3 μCi/min., **or**

or

B. Confirmed sample analyses for gaseous releases indicating total site release rates exceed:
   1. Noble gases >1000 mrem/year whole body, **or**
   2. Noble gases >6000 mrem/year skin, **or**
   3. I-131, I-133, H-3, and particulates with half-lives >8 days >3000 mrem/year to any organ (inhalation pathways only).

OR

EAL# 15.1.b Radiological liquid effluents exceed Technical Requirement Limits for instantaneous release as indicated by:

Report of radiological liquid effluent exceeding Technical Requirement Limits. This includes effluent sources such as Service Water or RHR Service Water Loops A or B.

(CONTINUED ON NEXT PAGE)

15 - RADIOLOGICAL EFFLUENT (continued)

ALERT

EAL# 15.2.a Radiological gaseous effluents exceed 200 times the Technical Requirement Limits for 15 minutes or longer as indicated by:

(A *or* B)

A. Valid Building Vent Stack Monitoring System (SPING) indications on Panel 0C630 or 0C677.
   1. Noble gases >1.70E+8 μCi/min., **or**
   2. I-131 >2.08E+4 μCi/min., **or**
   3. Particulate >1.54E+5 μCi/min.

or

B. Confirmed sample analyses for gaseous releases indicating total site release rates exceed:
   1. Noble gases >1.0E+5 mrem/year whole body, **or**
   2. Noble gases >6.0E+5 mrem/year skin, **or**
   3. I-131, I-133, H-3, and particulates with half-lives >8 days >3.0E+5 mrem/year to any organ (inhalation pathways only).

OR

EAL# 15.2.b Radiological liquid effluents exceed 10 times Technical Requirement Limits for instantaneous release as indicated by:

Report of radiological liquid effluent release exceeding 10 times Technical Requirement Limits. This includes effluent sources such as Service Water or RHR Service Water Loops A or B.

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15 - RADIOLOGICAL EFFLUENT (continued)

SITE AREA EMERGENCY

EAL# 15.3 Radiological effluent corresponds to greater than 50 mrem W.B.(1) or 250 mrem thyroid(2) for a half-hour or 500 mrem W.B.(1) or 2500 mrem thyroid(2) for 2 minutes at the emergency plan boundary as indicated by:

OR

Emergency plan boundary dose is projected to exceed 500 mrem W.B. (1) within 1 hour as indicated by:

(A *or* B)

A. Dose projections based on:
   (1 *or* 2)
   1. Building vent stack monitoring system indications on Panel 0C630 or 0C677.

   or

   2. Field monitoring data onsite or offsite.

or

B. Dose projections based on:
   (1 *or* 2)
   1. Building vent stack monitoring system indications on Panel 0C630 or 0C677.

   or

   2. Field monitoring data onsite or offsite

(CONTINUED ON NEXT PAGE)

(1) The sum of the Effective Dose Equivalent resulting from the exposure to external sources and the Committed Effective Dose Equivalent incurred from all significant inhalation pathways during the early phase.

(2) Committed Dose Equivalent to the thyroid from radioiodine.

15 - RADIOLOGICAL EFFLUENT (continued)

GENERAL EMERGENCY

EAL# 15.4 Radiological effluent release corresponds to 1 rem whole body(1) or 5 rem(2) thyroid at the emergency plan boundary as indicated by:

OR

Offsite doses are projected to exceed 1 rem whole body(1) or 5 rem thyroid(2) due to the event as indicated by:

(A *or* B)

A. Dose projections based on:
   (1 *or* 2 *or* 3)
   1. Building Vent Stack Monitoring System indication on Panel 0C630 or 0C677.

   or

   2. Field monitoring data onsite or offsite.

   or

   3. In-plant conditions.

or

B. Dose projections based on:
   (1 *or* 2 *or* 3)
   1. Building Vent Stack Monitoring System indication on Panel 0C630 or 0C677.

   or

   2. Field monitoring data onsite or offsite.

   or

   3. In-plant conditions.

(1) The sum of the Effective Dose Equivalent resulting from the exposure to external sources and the Committed Effective Dose Equivalent incurred from all significant inhalation pathways during the early phase.

(2) Committed Dose Equivalent to the thyroid from radioiodine.

16 - SECURITY EVENT

UNUSUAL EVENT

EAL# 16.1 Security threat or attempted entry or attempted sabotage as indicated by:

(A *or* B *or* C)

A. A report from Security of a security threat, attempted entry, or attempted sabotage of the owner controlled area adjacent to the site.

or

B. Any attempted act of sabotage which is deemed legitimate in the judgment of the SHIFT SUPERVISOR/EMERGENCY DIRECTOR, and affects plant operation.

or

C. A site specific credible security threat notification.

ALERT

EAL# 16.2 Ongoing Security Compromise as indicated by:

(A *or* B)

A. A report from Security that a security compromise is at the site but no penetration of protected areas has occurred.

or

B. Any act of sabotage which results in an actual or potential substantial degradation of the level of safety of the plant as judged by the SHIFT SUPERVISOR/EMERGENCY DIRECTOR.

SITE AREA EMERGENCY

EAL# 16.3 An ongoing adversary event threatens imminent loss of physical control of plant as indicated by:

(A *or* B)

A. Report from Security that the security of the plant vital area is threatened by unauthorized (forcible) entry into the protected area.

or

B. Any act of sabotage which results in actual or likely major failures of plant functions needed for protection of the public as judged by the SHIFT SUPERVISOR/EMERGENCY DIRECTOR.

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16 - SECURITY EVENT (continued)

GENERAL EMERGENCY

EAL# 16.4 Loss of physical control of facilities as indicated by:

(A *or* B)

A. Report from Security that a loss of physical control of plant vital areas has occurred.

or

B. Any act of sabotage which results in imminent significant cladding failure or fuel melting with a potential for loss of containment integrity or the potential for release of significant amounts of radioactivity in a short time as judged by the SHIFT SUPERVISOR/EMERGENCY DIRECTOR.

17 - SPENT FUEL RELATED INCIDENT

UNUSUAL EVENT

EAL# 17.1 Unanticipated or unplanned concentrations of airborne activity exist in normally accessible areas, which is not due to planned maintenance activities, as indicated by:

Concentrations exceed 500 times the DAC values of 10CFR20 Appendix B, Table I values for a single isotope, or full multiple isotopes where

$$\frac{C_A}{DAC_A} + \frac{C_B}{DAC_B} + \frac{C_C}{DAC_C} \ldots \frac{C_N}{DAC_N} \geq 500$$

ALERT

EAL# 17.2 Unexpected in-plant high radiation levels or airborne contamination which indicates a severe fuel handling accident as indicated by:

Refuel floor area radiation monitor reading 1000 times normal annunciation on Panel 1C601 (2C601) or indication on Panel 1C600 (2C600).

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17 - SPENT FUEL RELATED INCIDENT (continued)

SITE AREA EMERGENCY

EAL# 17.3.a Major damage to irradiated fuel with actual or clear potential for significant release of radioactive material to the environment as indicated by:

(A *and* B)

A. Dropping, bumping, or otherwise rough handling of a new **OR** irradiated fuel bundle with irradiated fuel in the pool.

and

B. (1 *or* 2)

1. Refueling floor area radiation monitor reading 1000 times normal annunciation on Panel 1C601 (2C601) or indication on Panel 1C600 (2C600).

or

2. Reactor Building vent stack monitoring system high radiation annunciation or indication on Panel 0C630 or 0C677.

OR

EAL# 17.3.b Damage to irradiated fuel due to uncontrolled decrease in the fuel pool level to below the level of the fuel as indicated by:

(A *and* B)

A. (1 *or* 2)

1. Uncovering of irradiated fuel confirmation by verification of significant leakage from spent fuel pool.

or

2. Visual observation of water level below irradiated fuel in the pool.

and

B. (1 *or* 2)

1. Refueling floor area radiation monitor annunciation on Panel 1C651 (2C651) or indication on Panel 1C600 (2C600).

or

2. Reactor Building vent stack monitoring system high radiation annunciation or indication on Panel 0C630 or 0C677.

GENERAL EMERGENCY

EAL# 17.4 None.

18 - STEAM LINE BREAK

UNUSUAL EVENT

EAL# 18.1 None.

ALERT

EAL# 18.2 MSIV malfunction causing leakage as indicated by:

(A *and* B)
A. Valid MSIV closure signal or indication on Panel 1C601 (2C601).
and
B. (1 *or* 2)
1. Valid Main Steam Line flow indication on Panel 1C652 (2C652).
or
2. Valid Main Steam Line radiation indication on Panel 1C600 (2C600).

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18 - STEAM LINE BREAK (continued)

SITE AREA EMERGENCY

EAL# 18.3 Steam line break occurs outside of containment without isolation as indicated by:

(A *or* B *or* C *or* D)

A. (1 *and* 2)
   1. Failure of both MSIVs in the line with the leak to close as indicated by position indication on Panel 1C601 (2C601).

   and

   2. (a *or* b)
      a. High MSL flow annunciation on Panel 1C601 (2C601) or indication on Panel 1C652 (2C652).

      or

      b. Other indication of main steam leakage outside containment.

or

B. (1 *and* 2)
   1. Failure of RCIC steam isolation valves HV-F008 and HV-F007 to close as indicated on Panel 1C601 (2C601).

   and

   2. (a *or* b *or* c *or* d *or* e *or* f)
      a. RCIC steamline pipe routing area high temperature annunciation on Panel 1C601 (2C601), or indication on Panel 1C614 (2C614).

      or

      b. RCIC equipment area high temperature annunciation on Panel 1C601 (2C601) or indication on Panel 1C614 (2C614).

      or

      c. RCIC steamline high flow annunciation on Panel 1C601 (2C601).

      or

      d. RCIC steamline tunnel ventilation high delta temperature annunciation on Panel 1C601 (2C601).

      or

      e. RCIC turbine exhaust diaphragm high pressure annunciation on Panel 1C601 (2C601).

      or

      f. Other indication of steam leakage from the RCIC system.

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18 - STEAM LINE BREAK (continued)

SITE AREA EMERGENCY (continued)

or

C. (1 *and* 2)

1. Failure of HPCI steam isolation valves HV-F002 and HV-F003 to close as indicated by position indicator on Panel 1C601 (2C601).

and

2. (a *or* b *or* c *or* d *or* e *or* f)
   a. HPCI steamline pipe routing area high temperature annunciation on Panel 1C601 (2C601), or indication on Panel 1C614 (2C614).

   or

   b. HPCI equipment area high temperature annunciation on Panel 1C601 (2C601) or indication on Panel 1C614 (2C614).

   or

   c. HPCI steamline high flow annunciation on Panel 1C601 (2C601).

   or

   d. HPCI steamline tunnel ventilation high delta temperature annunciation on Panel 1C601 (2C601).

   or

   e. HPCI turbine exhaust diaphragm high pressure annunciation on Panel 1C601 (2C601).

   or

   f. Other indication of steam leakage from the HPCI system.

or

D. Any other un-isolatable steam line breaks.

GENERAL EMERGENCY

EAL# 18.4 None.

19 - TOXIC/FLAMMABLE GASES

UNUSUAL EVENT

EAL# 19.1 Nearby or onsite release of potentially harmful quantifies of toxic or flammable material as indicated by:

Visual observation or notification received by the control room operator.

ALERT

EAL# 19.2 Entry of toxic or flammable gases into the facility, with subsequent habitability problem as indicated by:

Visual observation, direct measurement, or notification received by the control room operator.

SITE AREA EMERGENCY

EAL# 19.3 Toxic or flammable gases enter vital areas, restricting access and restricted access constitutes a safety problem, as determined by:

(A *and* B)
A. Shift Supervisor's evaluation.
and
B. Visual observation, direct measurement, or notification received by control room operator.

GENERAL EMERGENCY

EAL# 19.4 None.

20 - TECHNICAL SPECIFICATION SAFETY LIMIT

UNUSUAL EVENT

EAL# 20.1 Abnormal occurrences which result in operator complying with any of the Technical Specification SAFETY LIMIT ACTION statements indicated by:

(A *or* B *or* C *or* D)

A. Exceeding THERMAL POWER, low pressure or low flow safety limit 2.1.1.1.

or

B. Exceeding THERMAL POWER, high pressure and high flow safety limit 2.1.1.2.

or

C. Exceeding REACTOR VESSEL WATER LEVEL safety limit 2.1.1.3.

or

D. Exceeding REACTOR COOLANT SYSTEM PRESSURE safety limit 2.1.2.

ALERT

EAL# 20.2 None.

SITE AREA EMERGENCY

EAL# 20.3 None.

GENERAL EMERGENCY

EAL# 20.4 None.

21 – DRY FUEL STORAGE

UNUSUAL EVENT

EAL# 21.1.a. Situations are occurring or have occurred during the transport of the irradiated spent fuel to the onsite storage facility, which jeopardize the integrity of the spent fuel or its container as indicated by:

(A *or* B)

A. Radiological readings exceed 2 R/hour at the external surface of any transfer cask or horizontal storage module.

or

B. Radiological readings exceed 1 R/hour one foot away from the external surface of any transfer cask or horizontal storage module.

OR

EAL# 21.1.b. Situations are occurring or have occurred at the irradiated spent fuel storage facility, which jeopardize the integrity of the dry cask storage system as indicated by:

(A *or* B)

A. Radiological readings exceed 2 R/hour at the external surface of any transfer cask or horizontal storage module.

or

B. Radiological readings exceed 1 R/hour one foot away from the external surface of any transfer cask or horizontal storage module.

ALERT

EAL# 21.2 None.

SITE AREA EMERGENCY

EAL# 21.3 None.

GENERAL EMERGENCY

EAL# 21.4 None.