UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUN 25 1985

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NPF-21, WPPSS NUCLEAR PROJECT NO. 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 11 to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated May 16, 1985.

This action amends the WNP-2 Technical Specifications Emergency Core Cooling System Activation Instrumentation, Tables 3.3.3-1, 3.3.3-2 and 4.3.3.1-1 to remove the Automatic Depressurization System's (ADS) high drywell pressure instrumentation and add manual inhibit switches to the ADS logic.

A copy of the related safety evaluation supporting Amendment No. 11 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,

Walter R. Butler

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 11 to Facility Operating License NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. G. C. Sorensen, Manager
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21
Amendment No. 11

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee) dated May 16, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-21 is amended to revise the Technical Specifications as indicated in the attachments to this amendment and paragraph 2.C.(2) of Facility Operating License NPF-21 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 11, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to Technical Specifications

Date of Issuance: JUN 25 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 11
FACILITY OPERATING LICENSE NO. NPF-21
DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE	INSERT
3/4 3-26	3/4 3-26
3/4 3-27	3/4 3-27
3/4 3-30	3/4 3-30
3/4 3-31	3/4 3-31
3/4 3-34	3/4 3-34
3/4 3-35	3/4 3-35

TABLE 3.3.3-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM[a]	APPLICABLE OPERATIONAL CONDITIONS	ACTION
A. DIVISION I TRIP SYSTEM			
1. RHR-A (LPCI MODE) & LPCS SYSTEM			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2	1, 2, 3	30
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	1	1, 2, 3, 4*, 5*	31
d. Reactor Vessel Pressure-Low (LPCS Permissive)	1	1, 2, 3, 4*, 5*	32 33
e. Reactor Vessel Pressure-Low (LPCI Permissive)	1	1, 2, 3, 4*, 5*	32 33
f. LPCI Pump A Start Time Delay Relay	1	1, 2, 3, 4*, 5*	32
g. LPCI Pump A Discharge Flow-Low (Minimum Flow)	1	1, 2, 3, 4*, 5*	31
h. Manual Initiation	1/division	1, 2, 3, 4*, 5*	34
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"#			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3	30
b. ADS Timer	1	1, 2, 3	32
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	32
d. LPCS Pump Discharge Pressure-High (Pump Running)	2	1, 2, 3	32
e. LPCI Pump A Discharge Pressure-High (Pump Running)	2	1, 2, 3	32
f. Manual Initiation	2/division	1, 2, 3	35
g. Inhibit Switch	1/division	1, 2, 3	35

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM(a)	APPLICABLE OPERATIONAL CONDITIONS	ACTION
B. DIVISION 2 TRIP SYSTEM			
1. RHR B and C (LPCI MODE)			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2	1, 2, 3	30
c. Reactor Vessel Pressure-Low (LPCI Permissive)	1/valve	1, 2, 3,	32
		4*, 5*	33
d. LPCI Pump B Start Time Delay Relay	1	1, 2, 3, 4*, 5*	32
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	1/pump	1, 2, 3, 4*, 5*	31
f. Manual Initiation	1/division	1, 2, 3, 4*, 5*	34
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"#			
a. Reactor Vessel Water Level - Low Low Low, Level 1	2	1, 2, 3	30
b. ADS Timer	1	1, 2, 3	32
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	32
d. LPCI Pump B and C Discharge Pressure - High (Pump Running)	2/pump	1, 2, 3	32
e. Manual Initiation	2/division	1, 2, 3	35
f. Inhibit Switch	1/division	1, 2, 3	35

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
A. DIVISION 1 TRIP SYSTEM		
1. RHR-A (LPCI MODE) AND LPCS SYSTEM		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. Drywell Pressure - High	≤ 1.65 psig	≤ 1.85 psig
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	≥ 770 gpm	≤ 900 gpm
d. Reactor Vessel Pressure-Low (LPCS Permissive)	≥ 470 psig, decreasing	≥ 450 psig, decreasing
e. Reactor Vessel Pressure-Low (LPCI Permissive)	≥ 470 psig, decreasing	≥ 450 psig, decreasing
f. LPCI Pump A Start Time Delay Relay	≤ 5 seconds	≤ 6 seconds
g. LPCI Pump A Discharge Flow-Low (Minimum Flow)	≥ 800 gpm	≥ 650 gpm
h. Manual Initiation	N.A.	N.A.
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"		
a. Reactor Vessel Water Level - Low Low Low, Level 1	≥ -129 inches*	≥ -136 inches
b. ADS Timer	≤ 105 seconds	≤ 117 seconds
c. Reactor Vessel Water Level-Low, Level 3 (Permissive)	≥ 13.0 inches*	≥ 11 inches
d. LPCS Pump Discharge Pressure-High (Pump Running)	≥ 145 psig, increasing	≥ 125 psig, increasing
e. LPCI Pump A Discharge Pressure-High (Pump Running)	≥ 125 psig, increasing	≥ 115 psig, increasing
f. Manual Initiation	N.A.	N.A.
g. Inhibit Switch	N.A.	N.A.

TABLE 3.3.3-2 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
B. DIVISION 2 TRIP SYSTEM		
1. RHR B AND C (LPCI MODE)		
a. Reactor Vessel Water Level - Low Low Low, Level 1	$\geq$ -129 inches*	$\geq$ -136 inches
b. Drywell Pressure - High	$\leq$ 1.65 psig	$\leq$ 1.85 psig
c. Reactor Vessel Pressure-Low (LPCI Permissive)	$\geq$ 470 psig, decreasing	$\geq$ 450 psig, decreasing
d. LPCI Pump B Start Time Delay Relay	$\leq$ 5 seconds	$\leq$ 6 seconds
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	$\geq$ 800 gpm	$\geq$ 650 gpm
f. Manual Initiation	N.A.	N.A.
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"		
a. Reactor Vessel Water Level - Low Low Low, Level 1	$\geq$ -129 inches*	$\geq$ -136 inches
b. ADS Timer	$\leq$ 105 seconds	$\leq$ 117 seconds
c. Reactor Vessel Water Level-Low, Level 3 (Permissive)	$\geq$ 13.0 inches*	$\geq$ 11 inches
d. LPCI Pump B and C Discharge Pressure-High (Pump Running)	$\geq$ 125 psig, increasing	$\geq$ 115 psig, increasing
e. Manual Initiation	N.A.	N.A.
f. Inhibit Switch	N.A.	N.A.

TABLE 4.3.3.1-1

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
A. DIVISION I TRIP SYSTEM				
1. RHR-A (LPCI MODE) AND LPCS SYSTEM				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	N.A.	M	R	1, 2, 3
c. LPCS Pump Discharge Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
d. Reactor Vessel Pressure-Low (LPCS Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
e. Reactor Vessel Pressure-Low (LPCI Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
f. LPCI Pump A Start Time Delay Relay	N.A.	M	Q	1, 2, 3, 4*, 5*
g. LPCI Pump A Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
h. Manual Initiation	N.A.	R	N.A.	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3
b. ADS Timer	N.A.	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	S	M	R	1, 2, 3
d. LPCS Pump Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
e. LPCI Pump A Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
f. Manual Initiation	N.A.	R	N.A.	1, 2, 3
g. Inhibit Switch	N.A.	M	N.A.	1, 2, 3

TABLE 4.3.3.1-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
B. DIVISION 2 TRIP SYSTEM				
1. RHR B AND C (LPCI MODE)				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3, 4*, 5*
b. Drywell Pressure - High	N.A.	M	R	1, 2, 3
c. Reactor Vessel Pressure-Low (LPCI Permissive)	N.A.	M	R	1, 2, 3, 4*, 5*
d. LPCI Pump B Start Time Delay Relay	N.A.	M	Q	1, 2, 3, 4*, 5*
e. LPCI Pump Discharge Flow-Low (Minimum Flow)	N.A.	M	R	1, 2, 3, 4*, 5*
f. Manual Initiation	N.A.	R	N.A.	1, 2, 3, 4*, 5*
2. AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"#				
a. Reactor Vessel Water Level - Low Low Low, Level 1	S	M	R	1, 2, 3
b. ADS Timer	N.A.	M	Q	1, 2, 3
c. Reactor Vessel Water Level - Low, Level 3 (Permissive)	S	M	R	1, 2, 3
d. LPCI Pump B and C Discharge Pressure-High (Pump Running)	N.A.	M	R	1, 2, 3
e. Manual Initiation	N.A.	R	N.A.	1, 2, 3
f. Inhibit Switch	N.A.	N	N.A.	1, 2, 3

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 11 TO NPF-21

WPPSS NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

INTRODUCTION

By letter dated May 16, 1985, the licensee requested an amendment to the Technical Specifications Sections 3.3.3 and 4.3.3.1 of the WNP-2 license NPF-21.

EVALUATION

As currently installed, the automatic depressurization system (ADS), through selected safety/relief valves, functions as a backup to the operation of the high pressure coolant systems. The ADS depressurizes the vessel so that low pressure systems may inject water into the reactor vessel. The ADS is typically activated automatically upon coincident signals of low water level in the reactor vessel, high drywell pressure, and any low pressure ECCS pump running. A time delay of approximately two minutes after receipt of the coincident signals allows time for the automatic blowdown to be bypassed manually if the operator believes the signals are erroneous or if the water level can be restored.

For transient and accident events which do not directly produce a high drywell pressure signal (e.g., stuck open relief valve or steam line break outside containment) and are degraded by a loss of high pressure coolant systems, manual actuation of the ADS is required to provide adequate core cooling. An important consideration is that proposed modifications to the ADS logic should be such that operator actions which may be required during an ATWS should not be complicated by the ADS.

As a result of the accident at Three Mile Island, NUREG-0737 addressed this difficulty as Item II.K.3.18 which resulted in a discussion in Supplement 4 to the WNP-2 Safety Evaluation Report (NUREG-0892). Section 6.3 of the WNP-2 SSER 4 is quoted in its entirety as follows:

"6.3 Emergency Core Cooling System

6.3.6 TMI Action Items

II.K.3.18 Modification to ADS Logic

In a letter dated July 26, 1983, the applicant committed to modifying the automatic depressurization system (ADS). The design modification will bypass the high drywell pressure trip portion of the existing ADS logic and add a manually operated inhibit switch. This modification is Option 2 of the BWR Owners Group Report (Dente, 1982) and has been approved by the staff (Houston, 1983). The applicant's conceptual design is acceptable with the following conditions:

(1) Use of the manual inhibit switch must be addressed in the plant emergency procedures.

(2) Surveillance requirements for the inhibit switch must be included in the plant Technical specifications.

(3) The modifications must be complete before startup following the first refueling outage."

As indicated, the Supply System has endorsed the BWR Owners Group recommendations and elected to implement Option 2 as their specific resolution of the TMI Action Item II.K.3.18. The three conditions imposed by the staff have also been met:

(1) WNP-2 plant emergency procedures have been modified to incorporate use of the manual inhibit switch. The May 16, 1985 request specifically expresses their intent to incorporate the use of the inhibit switch into the emergency procedures and the staff has been informed by telephone that the emergency procedure modifications have been made.

(2) Surveillance requirements for the inhibit switch have been included in Table 4.3.3.1-1 as amended.

(3) The modifications will be complete before startup following the current maintenance outage which is approximately one year before the first scheduled refueling outage.

Based on the foregoing consideration, the staff finds the proposed modifications to the WNP-2 Automatic Depressurization System to be acceptable.

FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; (2) create the possibility of a new or different kind of accident from an accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The licensee has determined and the NRC staff agrees that the requested amendment per 10 CFR 50.92 does not:

1) Involve a significant increase in the probability or consequences of an accident previously evaluated because the proposed changes provide additional assurance of adequate core cooling by extending the operation of the ADS to encompass additional accident and transient conditions which do not directly produce a high drywell pressure signal. Thus the accident probability is actually decreased while the accident consequences are unchanged.

2) Create the possibility of a new or different kind of accident than previously evaluated because no new accident scenario is created by changing only the control logic for a safety system.

3) Involve a significant reduction in the margin of safety because the criteria for the performance of the ADS have not been changed and the control logic modifications extend the capability for the system to respond to additional accident scenarios not previously considered.

Accordingly, the Commission has determined that this amendment involves no significant hazards consideration.

BASIS FOR EMERGENCY SITUATION

This amendment is being issued on an emergency basis. Restart of the WNP-2 power plant is now scheduled for June 25, 1985. The staff has reviewed and evaluated the Supply System's request for an emergency circumstance (letter G02-85-302 from Sorensen, Supply System, to Butler, NRC, dated June 11, 1985) and agrees that the Supply System has set out an adequate explanation why this emergency situation occurred and why it could not avoid this situation.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to the requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has determined that this amendment involves no significant hazards consideration. Accordingly, this amendment meets the

eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: JUN 25 1985

Docket No. 50-397

JUN 25 1985

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NPF-21, WPPSS NUCLEAR PROJECT NO. 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 11 to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated May 16, 1985.

This action amends the WNP-2 Technical Specifications Emergency Core Cooling System Activation Instrumentation, Tables 3.3.3-1, 3.3.3-2 and 4.3.3.1-1 to remove the Automatic Depressurization System's (ADS) high drywell pressure instrumentation and add manual inhibit switches to the ADS logic.

A copy of the related safety evaluation supporting Amendment No. 11 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,
Original signed by:

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosures:
1. Amendment No. 11 to Facility Operating License NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

*Previously concurred:

WB

DL:LB#2	DL:LB#2	OELD	DL:LB#2	AD/L/DL
*JBradfute:lam	EPayton	*WPaton	WButler	TMNovak
06/19/85	06/21/85	06/2085	06/21/85	06/25/85

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by

Walter R. Butler, Chief
Licensing Branch No. 2
Division of Licensing

Enclosure:
Changes to Technical Specifications

Date of Issuance: JUN 25 1985

DL:LB#2	DL:LB#2	DSI:RSB	OELD	DL:LB#2	AD/L/DL
EHylton:lam	JBradfute	BSheron	WPaton	Butler	TNovak
06/ /85	06/19/85	06/21/85	06/ /85	06/19/85	06/ /85

Issuance of Amendment No. 11 to Facility Operating License No. NPF-21
WPPSS Nuclear Project No. 2

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