



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

JUN 25 1985

Docket No. 50-397

Mr. G. C. Sorensen, Manager  
Regulatory Programs  
Washington Public Power Supply System  
P. O. Box 968  
3000 George Washington Way  
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 12 TO FACILITY OPERATING LICENSE  
NPF-21, WPPSS NUCLEAR PROJECT NO. 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 12 to Facility Operating License NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington. This amendment is in response to your letter dated March 13, 1985.

This action amends the WNP-2 Technical Specifications, Isolation Actuation Instrumentation, Tables 3.3.2-1, 3.3.2-2, 3.3.2-3 and 4.3.2.1-1 to change some of the Isolation Actuation Instrumentation, their trip setpoints and their allowable values. These changes are made to reflect more realistic values as determined by actual plant operation and as required by the Technical Specifications themselves. In addition, this amendment will change the delta temperature-high signal to a temperature-high signal for isolation actuation from the Residual Heat Removal (RHR) heat exchanger area and correct an inconsistency in the quality assurance record retention requirements.

A copy of the related safety evaluation supporting Amendment No. 12 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,

A handwritten signature in cursive script that reads "Walter R. Butler".

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosures:

1. Amendment No. 12 to Facility Operating License NPF-21
2. Safety Evaluation

cc w/enclosures:  
See next page

8507030039 850625  
PDR ADDCK 05000397  
P PDR

Mr. G. C. Sorensen, Manager  
Washington Public Power Supply System

WPPSS Nuclear Project No. 2  
(WNP-2)

cc: Nicholas Reynolds, Esquire  
Bishop, Cook, Liberman,  
Purcell & Reynolds  
1200 Seventeenth Street, N. W.  
Washington, D. C. 20036

Mr. John B. Martin  
U. S. NRC, Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596

Mr. G. E. Doupe, Esquire  
Washington Public Power Supply System  
P. O. Box 968  
3000 George Washington Way  
Richland, Washington 99532

Curtis Eschels, Chairman  
Energy Facility Site Evaluation Council  
Mail Stop PY-11  
Olympia, Washington 98504

P. L. Powell, Licensing Manager  
Washington Public Power Supply System  
P. O. Box 968  
Richland, Washington 99352

Mr. W. G. Conn, SR. N/M Group Supervisor  
Burns and Roe, Incorporated  
c/o Washington Public Power Supply System  
P. O. Box 968 MD 994E  
Richland, Washington 99352

R. B. Glasscock, Director  
Licensing and Assurance  
Washington Public Power Supply System  
P. O. Box 968, MD 280  
Richland, Washington 99352

Mr. C. M. Powers  
WNP-2 Plant Manager  
Washington Public Power Supply System  
P. O. Box 968 MD 927  
Richland, Washington 99352



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21  
Amendment No. 12

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment filed by the Washington Public Power Supply System (the Supply System, also the licensee) dated March 13, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. NPF-21 is amended to revise the Technical Specifications as indicated in the attachments to this amendment and paragraph 2.C.(2) of Facility Operating License NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8507030043 850625  
PDR ADOCK 05000397  
P PDR

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Handwritten signature of Walter R. Butler in cursive script.

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to Technical Specifications

Date of Issuance: **JUN 25 1985**

ATTACHMENT TO LICENSE AMENDMENT NO. 12  
FACILITY OPERATING LICENSE NO. NPF-21  
DOCKET NO. 50-397

Replace the following pages of the Appendix "A" Technical Specifications with enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

REMOVE

3/4 3-13  
3/4 3-14  
3/4 3-16  
3/4 3-17  
3/4 3-18  
3/4 3-20  
3/4 3-24  
6-23

INSERT

3/4 3-13  
3/4 3-14  
3/4 3-16  
3/4 3-17  
3/4 3-18  
3/4 3-20  
3/4 3-24  
6-23

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
3. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>				
a. $\Delta$ Flow - High	7	1	1, 2, 3	22
b. Heat Exchanger Area Temperature - High	7	1	1, 2, 3	22
c. Heat Exchanger Area Ventilation $\Delta$ Temp. - High	7	1	1, 2, 3	22
d. Pump Area Temperature - High				
1) Pump Room A	7	1	1, 2, 3	22
2) Pump Room B	7	1	1, 2, 3	22
e. Pump Area Ventilation $\Delta$ Temp. - High				
1) Pump Room A	7	1	1, 2, 3	22
2) Pump Room B	7	1	1, 2, 3	22
f. SLCS Initiation	7(f)	N.A.	1, 2, 3	22
g. Reactor Vessel Water Level - Low Low, Level 2	7	2	1, 2, 3	22
h. RWCU/RCIC Line Routing Area Temperature - High	7	1	1, 2, 3	22
i. RWCU Line Routing Area Temperature - High				
Room 509	7	1	1, 2, 3	22
Room 511	7	1	1, 2, 3	22
Room 408	7	1	1, 2, 3	22
Room 409	7	1	1, 2, 3	22
j. Manual Initiation	7	1/group	1, 2, 3	24

TABLE 3.3.2-1 (Continued)

<u>ISOLATION ACTUATION INSTRUMENTATION</u>				
<u>TRIP FUNCTION</u>	<u>VALVE GROUPS OPERATED BY SIGNAL</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
<u>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>				
a. RCIC Steam Line Flow - High	8	1	1, 2, 3	22
b. RCIC/RHR Steam Line Flow - High	8	1	1, 2, 3	22
c. RCIC Steam Supply Pressure - Low	8, 9	2	1, 2, 3	22
d. RCIC Turbine Exhaust Diaphragm Pressure - High	8	2	1, 2, 3	22
e. RCIC Equipment Room Temperature - High	8	1	1, 2, 3	22
f. RCIC Equipment Room $\Delta$ Temperature - High	8	1	1, 2, 3	22
g. RWCU/RCIC Steam Line Routing Area Temperature - High	8	1	1, 2, 3	22
h. Drywell Pressure - High	9	2	1, 2, 3	22
i. Manual Initiation(h)	8	1	1, 2, 3	24
<u>5. RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 3	6	2	1, 2, 3	26
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	6	1	1, 2, 3	26
c. Equipment Area Temperature - High	6	1	1, 2, 3	26
d. Equipment Area Ventilation $\Delta$ Temp. - High	6	1	1, 2, 3	26
e. Shutdown Cooling Suction Flow Rate - High	6	1	1, 2, 3	26
f. RHR Heat Exchanger Area Temperature - High				
Room 606	6	1	1, 2, 3	26
Room 507	6	1	1, 2, 3	26
Room 605	6	1	1, 2, 3	26
Room 505	6	1	1, 2, 3	26
g. Manual Initiation	6	1/group	1, 2, 3	24

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<u>1. PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level		
1) Low, Level 3	> 13.0 inches*	> 11.0 inches
2) Low Low, Level 2	> -50 inches*	> -57 inches
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Main Steam Line		
1) Radiation - High	≤ 3.0 x full power background	≤ 3.6 x full power background
2) Pressure - Low	> 831 psig	> 811 psig
3) Flow - High	< 105.5 psid	< 108 psid
d. Main Steam Line Tunnel Temperature - High	≤ 150°F	≤ 170°F
e. Main Steam Line Tunnel Δ Temperature - High	< 80°F	< 90°F
f. Condenser Vacuum - Low	> 23 inches Hg absolute pressure	> 24.5 inches Hg absolute pressure
g. Manual Initiation	N.A.	N.A.
<u>2. SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Building Vent Exhaust Plenum Radiation - High	< 13.0 mR/h	< 16.0 mR/h
b. Drywell Pressure - High	< 1.68 psig	< 1.88 psig
c. Reactor Vessel Water Level - Low Low, Level 2	> -50 inches*	> -57 inches
d. Manual Initiation	N.A.	N.A.



TABLE 3.3.2-2 (Continued)  
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
<b>3. REACTOR WATER CLEANUP SYSTEM ISOLATION</b>		
a. Δ Flow - High	≤ 58.5 gpm	≤ 65.5 gpm
b. Heat Exchanger Area Temperature - High	≤ 150°F	≤ 160°F
c. Heat Exchanger Area Ventilation Δ Temp. - High	≤ 60°F	≤ 70°F
d. Pump Area Temperature - High Pump Room A	≤ 160°F	≤ 180°F
Pump Room B	≤ 160°F	≤ 180°F
e. Pump Area Ventilation Δ Temp. - High Pump Room A	≤ 70°F	≤ 100°F
Pump Room B	≤ 70°F	≤ 100°F
f. SLCS Initiation	N.A.	N.A.
g. Reactor Vessel Water Level - Low Low, Level 2	≥ -50 inches*	≥ -57 inches
h. RWCU/RCIC Line Routing Area Temperature - High	≤ 160°F	≤ 180°F
i. RWCU Line Routing Area Temperature - High Room 409	≤ 160°F	≤ 175°F
Room 511	≤ 160°F	≤ 180°F
Room 408	≤ 160°F	≤ 180°F
Room 409	≤ 160°F	≤ 175°F
j. Manual Initiation	N.A.	N.A.
<b>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</b>		
a. RCIC Steam Line Flow - High	≤ 290% of rated flow	≤ 300% of rated flow
b. RHR/RCIC Steam Line Flow - High	≤ 101.5 inches H <sub>2</sub> O	≤ 107.5 inches H <sub>2</sub> O
c. RCIC Steam Supply Pressure - Low	≥ 62 psig	≥ 58 psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	≤ 10.0 psig	≤ 20.0 psig
e. RCIC Equipment Room Temperature - High	≤ 160°F	≤ 180°F

TABLE 3.3.2-2 (Continued)  
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u> (Continued)		
f. RCIC Equipment Room $\Delta$ Temperature - High	$\leq 50^{\circ}\text{F}$	$\leq 60^{\circ}\text{F}$
g. RWCU/RCIC Steam Line Routing Area Temperature - High	$\leq 160^{\circ}\text{F}$	$\leq 180^{\circ}\text{F}$
h. Drywell Pressure - High	$\leq 1.65$ psig	$\leq 1.85$ psig
i. Manual Initiation	N.A.	N.A.
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 3	$\geq 13.0$ inches*	$\geq 11.0$ inches
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	$\leq 125$ psig	$\leq 135$ psig
c. Equipment Area Temperature - High Pump Room A	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
Pump Room B	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
d. Equipment Area Ventilation $\Delta$ Temp. - High Pump Room A	$\leq 55^{\circ}\text{F}$	$\leq 70^{\circ}\text{F}$
Pump Room B	$\leq 55^{\circ}\text{F}$	$\leq 70^{\circ}\text{F}$
e. Shutdown Cooling Return Flow Rate - High	$\leq 174$ inches H <sub>2</sub> O	$\leq 183$ inches H <sub>2</sub> O
f. RHR Heat Exchanger Area Temperature - High Room 606	$\leq 130^{\circ}\text{F}$	$\leq 140^{\circ}\text{F}$
Room 507	$\leq 150^{\circ}\text{F}$	$\leq 160^{\circ}\text{F}$
Room 605	$\leq 140^{\circ}\text{F}$	$\leq 150^{\circ}\text{F}$
Room 505	$\leq 130^{\circ}\text{F}$	$\leq 140^{\circ}\text{F}$
g. Manual Initiation	N.A.	N.A.

TABLE NOTATIONS

\*See Bases Figure B 3/4 3-1.

TABLE 3.3.2-3 (Continued)

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
<u>4. REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>	
a. RCIC Steam Line Flow - High	< 13(a)
b. RHR/RCIC Steam Line Flow - High	< 13(a)
c. RCIC Steam Supply Pressure - Low	< 13(a)
d. RCIC Turbine Exhaust Diaphragm Pressure - High	N.A.
e. RCIC Equipment Room Temperature - High	N.A.
f. RCIC Equipment Room Δ Temperature - High	N.A.
g. RWCU/RCIC Steam Line Routing Area Temperature - High	N.A.
h. Drywell Pressure - High	N.A.
i. Manual Initiation	N.A.
<u>5. RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>	
a. Reactor Vessel Water Level - Low, Level 3	≤ 13(a)
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	N.A.
c. Equipment Area Temperature - High	N.A.
d. Equipment Area Ventilation Δ Temp. - High	N.A.
e. Shutdown Cooling Return Flow Rate - High	N.A.
f. RHR Heat Exchanger Area Temperature - High	N.A.
g. Manual Initiation	N.A.

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u>
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u> (Continued)				
g. RWCU/RCIC Steam Line Routing Area Temperature - High	S	M	R	1, 2, 3
h. Drywell Pressure - High	N.A.	M	R	1, 2, 3
i. Manual Initiation	N.A.	R	N.A.	1, 2, 3
5. <u>RHR SYSTEM SHUTDOWN COOLING MODE ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 3	S	M	R	1, 2, 3
b. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	N.A.	M	R	1, 2, 3
c. Equipment Area Temperature - High	S	M	R	1, 2, 3
d. Equipment Area Ventilation $\Delta$ Temp. - High	S	M	R	1, 2, 3
e. Shutdown Cooling Return Flow Rate - High	N.A.	M	R	1, 2, 3
f. RHR Heat Exchanger Area Temperature - High	S	M	R	1, 2, 3
g. Manual Initiation	N.A.	R	N.A.	1, 2, 3

TABLE NOTATIONS

- \* When reactor steam pressure  $\geq$  1037 psig and/or any turbine stop valve is open.
- \*\* When handling irradiated fuel in the secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- # During CORE ALTERATION and operations with a potential for draining the reactor vessel.

## ADMINISTRATIVE CONTROLS

---

### RECORD RETENTION (Continued)

- e. Records of changes made to the procedures required by Specification 6.8.1.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.

6.10.3 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report (FSAR).
- b. Records of new and irradiated fuel inventory, fuel transfers, and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7.1-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of inservice inspections performed pursuant to these Technical Specifications.
- i. Records of quality assurance activities required by the Operational Quality Assurance Manual not listed in Section 6.10.2.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the POC and the CNSRB.
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.4 including the date at which the service life commences and associated installation and maintenance records.
- m. Records of analysis required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION

AMENDMENT NO. 12 TO NPF-21

WPPSS NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

INTRODUCTION

By letter dated March 13, 1985, the licensee requested an amendment to the Technical Specifications, Isolation Actuation Instrumentation, Section 3/4 3.2 of the WNP-2 license NPF-21.

EVALUATION

Prior to full power operation, analyses were performed by the Supply System to determine a best estimate for the trip setpoint values for isolation actuation of several systems in both the primary and secondary containments. Similarly best estimates for allowable values of these parameters were also obtained. Specifically these systems are:

- the main steam line tunnel in the primary containment
- the exhaust plenum of the reactor building vent
- the reactor water coolant system
- the reactor core isolation cooling (RCIC) system
- the residual heat removal (RHR) system

When the Technical Specifications were written it was recognized by the staff that these values were preliminary and that they should be changed to reflect actual plant operating conditions.

These measurements of these parameters are used primarily to detect leaks and line breaks in various rooms and areas of the containments. An increase in air temperature is signaled in the control room and used as an initial indication that a high energy line leak has occurred. If this initial indication is not severe it is used as a basis for an inspection, evaluation and manual intervention as appropriate. If the measured signal exceeds the trip setpoint value, the signal automatically causes an isolation of the systems associated with the local area or room. Initially, conservative values of the leak detection setpoints were deliberately chosen to permit early warning and isolation for local leaks and pipe breaks.

When the Operating License was issued, the Technical Specifications included trip setpoints and allowable values that were based on these conservative engineering estimates with the expectation that these values would be adjusted following plant startup tests when actual ambient conditions were established. These values were so noted in the Technical Specifications themselves and more appropriate values were to be determined and submitted to the Commission. (See "TABLE NOTATIONS," page 3/4 3-18 of the Technical

8507030048 850625  
PDR ADDCK 05000397  
P PDR

Specifications). The Supply System has completed the necessary testing, measurements and analyses and has submitted the required values as a request for amendment to the Technical Specifications.

In addition, the amendment will change the delta temperature - high signal to a temperature - high signal for isolation actuation from the RHR Heat Exchanger Area and correct an inconsistency in the quality assurance record retention requirements. The substitution of a temperature measurement for a temperature difference measurement reflects an operational preference and is equally valid as an indication of a significant leak. The quality assurance record retention change is purely administrative and without safety implications.

The licensee has determined that no unresolved safety questions will result from these changes. The staff has reviewed this determination and the proposed changes to the Technical Specifications and conclude that they are acceptable.

#### FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; (2) create the possibility of a new or different kind of accident from an accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The licensee has determined and the NRC staff agrees that the requested amendment per 10 CFR 50.92 does not:

- 1) involve a significant increase in the probability or consequences of an accident previously evaluated because the updated temperature setpoints use the same leakage criteria as previously and do not alter the valve closure times, so accident probabilities and consequences are not affected; or
- 2) create the possibility of a new or different kind of accident than previously evaluated because this change introduces no new accident types nor changes any criteria; or
- 3) involve a significant reduction in a margin of safety because the same criteria for allowable leakage prior to high temperature trip have been used, so the margin of safety has been maintained.

Accordingly, the Commission has determined that this amendment involves no significant hazards consideration.

On June 18, 1985 the Commission published in local newspapers notice of its proposal to amend the Supply System's license. No public comment was received relative to this amendment. In addition, the State of Washington has been notified of the request for amendment by the Supply System and they indicated their concurrence by telephone on June 17, 1985.

### ENVIRONMENTAL CONSIDERATION

This amendment involves a change to the requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has determined that this amendment involves no significant hazards consideration. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: **JUN 25 1985**

Principal Contributor: J. Bradfute



3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Walter R. Butler, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Changes to Technical Specifications

Date of Issuance: JUN 25 1985

*EB*  
DL:LB#2  
EK/ston  
06/19/85

*BRADY*  
DL:LB#2  
JBradfute  
06/19/85

*WD Paton*  
OELD  
WPaton  
06/20/85

*WB*  
DL:LB#2  
Butler  
06/19/85

*AD/TL*  
AD/TL  
TNovak  
06/25/85

Issuance of Amendment No. 12 to Facility Operating License No. NPF-21  
WPPSS Nuclear Project No. 2

DISTRIBUTION

Docket File

NRC PDR  
Local PDR  
PRC System  
NSIC  
LB#2 Reading  
EHylton  
JBradfute  
TNovak  
JSaltzman, SAB  
Paton, ELD  
OMiles  
HDenton  
JRutberg  
AToalston  
WMiller, LFMB  
JPartlow  
EJordan  
BGrimes  
LHarman  
TBarrhart (4)  
EButcher