Docket Files BR 50-397

Docket No. 50-397

JUL 1 3 1984

Mr. G. C. Sorensen, Manager Regulatory Programs Washington Public Power Supply System P. O. Box 968 3000 George Washington Way Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: Issuance of Amendment No. 2 to Facility Operating

License No. NPF-21, WPPSS Nuclear Project No. 2

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. NPF-21 to the Washington Public Power Supply System for WPPSS Nuclear Project No. 2, located in Benton County near Richland, Washington.

The amendment is in response to your letter dated April 10, 1984. This amendment would delete the surveillance requirements b and d in paragraph 4.4.3.2.2 of the Technical Specification for certain reactor coolant system pressure isolation valves. These valves are identified in Table 3.4.3.2-1 of the Technical Specifications.

A copy of the related safety evaluation supporting Amendment No. 2 to Facility Operating License No. NPF-21 is enclosed.

Sincerely,

Original signed by

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Enclosures:

 Amendment No. 2 to Facility Operating License No. NPF-21

2. Safety Evaluation

cc: w/enclosures See next page

DL:LTM2/PM RANTuck:bdm DL:LR#2/LA ENVION 5/19/84 OELD ATTY WPaton 5/6/84

DL:LB#2/BC ASchwencer 5/ /84

8407230286 840713 PDR ADDCK 05000397 PDR Mr. G. C. Sorensen, Manager
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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

WPPSS NUCLEAR PROJECT NO.2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-21 Amendment No. 2

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment filed by the Washington Public Power Supply System (WPPSS, also the licensee), dated January 20, February 8, and April 10, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulation set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. The license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 2, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment was effective as of April 10, 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Attachment:

Changes to the Technical Specifications

Date of Issuance: JUL 1 3 1984

DL:LB##PM PAULUCK:bdm DL:LF#2/LA EH916on 5/93/84 OELD:ATTY WPaton 54 784

DL:LB#2/BC ASchwencer

7/13/84

ATTACHMENT TO LICENSE AMENDMENT NO. 2 FACILITY OPERATING LICENSE NO. NPF-21 DOCKET NO. 50-397

Replace the following page of the Appendix "A" Technical Specifications with enclosed page. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
3/4.4-10	3/4.4-10

SURVEILLANCE REQUIREMENTS

- 4.4.3.2.1 The reactor coolant system leakage shall be demonstrated to be within each of the above limits by:
 - a. Monitoring the primary containment atmospheric particulate and gaseous radioactivity at least once per 12 hours,
 - Monitoring the primary containment sump flow rate at least once per 12 hours,
 - c. Monitoring the reactor vessel head flange leak detection system at least once per 24 hours.
- 4.4.3.2.2 Each reactor coolant system pressure isolation valve specified in Table 3.4.3.2-1 shall be demonstrated OPERABLE by leak testing pursuant to Specification 4.0.5 and verifying the leakage of each valve to be within the specified limit:
 - a. At least once per 18 months.
 - b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve which could affect its leakage rate.

The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 3.

- 4.4.3.2.3 The high/low pressure interface valve leakage pressure monitors shall be demonstrated OPERABLE with alarm setpoints per Table 3.4.3.2-2 by performance of a:
 - a. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
 - b. CHANNEL CALIBRATION at least once per 18 months.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

JUL 1 3 1984

SAFETY EVALUATION
AMENDMENT NO. 2 TO NPF-21
WPPSS NUCLEAR PROJECT NO.2
DOCKET NO. 50-397

Introduction

In letters from G. C. Sorensen to A. Schwencer (References 1, 2 and 3), the Washington Public Power Supply System submitted a request for change in Technical Specification 3.4.3.2.

The proposed change will delete paragraphs b. and d. of Surveillance Requirement 4.4.3.2.2 pertaining to leak testing of pressure isolation valves (PIVs) for WNP-2. These paragraphs now require the PIVs to be leak tested (b) before going to hot shutdown during a cold shutdown of 72 hours or more if the PIVs have not been tested during the previous 9 months and (d) within 24 hours of valve actuation due to automatic or manual action or due to flow through the valve.

Evaluation

Based on our review of information in References 1, 2 and 3, we have determined that the licensee's proposed Technical Specification change is acceptable because the plant has the following features: (1) low pressure injection valve open permissives have been modified to actuate on low reactor pressure rather than delta pressure across the valves, (2) high/low pressure interface valve leakage pressure monitor alarms are available in the Main Control room which are surveillance tested periodically, (3) position indication on each interface valve is available in the Control room, and (4) narrow range suppression pool level indication is available which is sufficiently sensitive to detect gross system leakage. These features provide equal or greater assurance that the PIVs are in the closed and leak-tight position as the requirements of paragraphs (b) and (d) provide for other plants which do not have these control and automatic leakage verification features. Paragraphs (b) and (d), in effect, require the valves to be tested somewhat more frequently than every 18 months. With the four plant features outlined above, the PIV's are controlled and verified continually rather than at the intervals specified in paragraphs (b) and (d). Therefore, this Technical Specification change provides equal or more assurance of valve integrity and does not adversely affect the health and safety of the public.

Environmental Consideration

We have determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) this amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, creation of a new or different accident from any previously evaluated, or a significant reduction in a safety margin and, therefore, does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

References

- 1. Letter, G. C. Sorensen to A. Schwencer, "Operating License NPF-21, Request for License Amendment," dated January 20, 1984.
- 2. Letter, G. C. Sorensen to A. Schwencer, "License Amendment 10 CFR 50.92, Significant Hazards Analysis," dated February 8, 1984.
- 3. Letter, G. C. Sorensen to A. Schwencer, "Operating License NPF-21, Request for License Amendment," dated April 10, 1984 (G02-84-218).