July 8, 1999

Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. MA4601)

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. 125 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 12, 1999, as supplemented by letters dated May 11and June 30, 1999.

The amendment revises TS 3/4.7.5, Ultimate Heat Sink, by adding a new Action Statement to be used in the event that plant inlet water temperature exceeds 90°F. The amendment is effective only through September 30, 1999, and is only for the current TSs. The amendment is also limited to a maximum plant inlet water temperature of 94°F. The proposal to raise this temperature limit to 95°F will be addressed in a future letter.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original Signed By

Jack N. Donohew, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 125 to NPF-42
2. Safety Evaluation
cc w/encls: See next page
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DF-1

Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. MA4601)

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated January 12, 1999, as supplemented by letters dated May 11and June 30, 1999.

The amendment revises TS 3/4.7.5, Ultimate Heat Sink, by adding a new Action Statement to be used in the event that plant inlet water temperature exceeds 90°F. The amendment is effective only through September 30, 1999, and is only for the current TSs. The amendment is also limited to a maximum plant inlet water temperature of 94°F. The proposal to raise this temperature to 95°F will be addressed in a future letter.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Jack N. Donohew, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. to NPF-42
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| DATE | 7/6/99 | 7/ /99 | 7/7/99 | 7/7/99 | 7/08/99 | 7/ /99 |

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Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. MA4601)

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated January 12, 1999, as supplemented by letter dated May 11, 1999.

The amendment revises TS 3/4.7.5, Ultimate Heat Sink, by adding a new Action Statement to be used in the event that plant inlet water temperature exceeds 90°F.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Kristine M. Thomas, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. to NPF-42
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 8, 1999

Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. MA4601)

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. 125 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated January 12, 1999, as supplemented by letters dated May 11 and June 30, 1999.

The amendment revises TS 3/4.7.5, Ultimate Heat Sink, by adding a new Action Statement to be used in the event that plant inlet water temperature exceeds 90°F. The amendment is effective only through September 30, 1999, and is only for the current TSs. The amendment is also limited to a maximum plant inlet water temperature of 94°F. The proposal to raise this temperature limit to 95°F will be addressed in a future letter.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Jack N. Donohew

Jack N. Donohew, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 125 to NPF-42
2. Safety Evaluation

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Wolf Creek Nuclear Operating Corporation
P.O. Box 411
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U.S. Nuclear Regulatory Commission
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8201 NRC Road
Steedman, Missouri 65077-1032

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated January 12, 1999, as supplemented by letters dated May 11 and June 30, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

   2. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 125, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Stephen Dembek, Chief, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 8, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 125

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

| REMOVE | INSERT |
|---|---|
| 3/4 7-13 | 3/4 7-13 |

## 3/4.7.5 ULTIMATE HEAT SINK

### LIMITING CONDITION FOR OPERATION

3.7.5 The ultimate heat sink (UHS) shall be OPERABLE with:

a. The crest of the UHS dam below the rip rap cover and corresponding water level at or above elevation 1070 Mean Sea Level, USGS datum, and

b. The plant inlet water temperature of less than or equal to 90°F.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the requirements of the above specification not satisified, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

NOTE: Until September 30, 1999, the following ACTIONS supersede the above ACTION:

a. With the plant inlet water temperature >90°F but ≤94°F:

1. within one hour verify two trains of residual heat removal, two trains of component cooling water, and two trains of essential service water are OPERABLE,

2. verify at least once per hour that the plant inlet water temperature is ≤94°F, and

3. within 12 hours restore the plant inlet water temperature to 90°F or less.

4. With any of the above requirements not satisfied, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With the UHS inoperable for any reason other than temperature, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

4.7.5 The UHS shall be determined OPERABLE:

a. At least once per 24 hours by verifying the above required water temperature and water level to be within their limits, and

b. At least once per 12 months by verifying that the crest of the UHS dam below the rip rap cover is at or above elevation 1070 Mean Sea Level, USGS datum.

## 3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM

### LIMITING CONDITION FOR OPERATION

3.7.6 Two independent Control Room Emergency Ventilation Systems shall be OPERABLE.

APPLICABILITY: All MODES.

ACTION:

MODES 1, 2, 3 and 4:

With one Control Room Emergency Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5 and 6:

a. With one Control Room Emergency Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or initiate and maintain operation of the remaining OPERABLE Control Room Emergency Ventilation System in the recirculation mode. The provisions of Specification 3.0.4 are not applicable.

b. With both Control Room Emergency Ventilation Systems inoperable, or with the OPERABLE Control Room Emergency Ventilation System, required to be in the recirculation mode by ACTION a., not capable of being powered by an OPERABLE emergency power source, suspend all operations involving CORE ALTERATIONS or positive reactivity changes.

### SURVEILLANCE REQUIREMENTS

4.7.6 Each Control Room Emergency Ventilation System shall be demonstrated OPERABLE:

a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 84°F;

b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers of both the Filtration and Pressurization Systems and verifying that the Pressurization System operates for at least 10 continuous hours with the heaters operating;

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

## 1.0 INTRODUCTION

By letter dated January 12, 1999, as supplemented by letters dated May 11 and June 30, 1999, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (TSs), Appendix A to Facility Operating License No. NPF-42, for the Wolf Creek Generating Station (WCGS). The proposed changes would revise TS 3/4.7.5, Ultimate Heat Sink, by adding a new action statement to be used in the event that plant inlet water temperature exceeds 90°F. Specifically, the new action statement would allow, until September 30, 1999, continued operation of the plant with plant inlet water temperature between 90 and 94°F for up to 12 hours before requiring shutdown of the plant. The licensee will pursue a permanent change to the inlet water temperature limit through the Westinghouse Owners Group as a proposed change to the Standard Technical Specifications.

The licensee's proposed amendment is the same proposal that the licensee submitted on July 17, 1998, and the staff approved in Amendment 118 that was issued on July 18, 1998. The amendment was issued under emergency conditions, in accordance with 10 CFR 50.91(a)(5), because the cooling lake for WCGS exceeded 89°F and it was projected that the water temperature would exceed the TS limit of 90°F and the plant would have to unnecessarily shut down because of the then harsh meteorological conditions.

Amendment 118 was effective until September 30, 1998. In the letter of January 12, 1999, the licensee proposed a permanent change to TS 3/4.7.5; however, the staff concluded that the proposed change was generic in nature and requested that the licensee propose a limited duration amendment similar to that approved in Amendment 118. The licensee's modified proposals are in the letters of May 11 and June 30, 1999, both of which are of limited duration and would be effective only until September 30, 1999.

The May 11 and June 30, 1999, supplemental letters provided additional clarifying information, did not expand the scope of the application as originally noticed and did not change the staff's original proposed no significant hazards consideration determination published in the Federal Register on February 24, 1999 (64 FR 9203), except that in the June 30, 1999, letter the licensee proposed a maximum plant inlet water temperature of 95°F where the letters of January 12 and May 11, 1999, proposed only 94°F.

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The licensee's proposal that the staff is considering at this time is the plant inlet temperature limit of 94°F from the letters dated January 12 and May 11, 1999, and the action statements but not the temperature limit from the letter dated June 30, 1999. Thus, as limited, the letter of June 30, 1999, does not expand the scope of the application as originally noticed and does not change the staff's original proposed no significant hazards consideration determination published in the Federal Register on February 24, 1999 (64 FR 9203).

The current temperature of the cooling lake for WCGS is 85°F and rising. It is possible that the water temperature may exceed the TS limit of 90°F during the summer.

## 2.0 LICENSEE'S PROPOSAL

The proposed new action statement to TS 3/4.7.5 being considered by the staff is the following:

> NOTE: Until September 30, 1999, the following actions supersede the above action (of TS 3/4.7.5):
>
> a. With the plant inlet water temperature >90°F but <94°F:
>
> 1. Within one hour verify two trains of residual heat removal, two trains of component cooling water, and two trains of essential service water are operable,
>
> 2. Verify at least once per hour that the plant inlet temperature is <94°F, and
>
> 3. Within 12 hours restore the plant inlet water temperature to 90°F or less.
>
> 4. With any of the above requirements not satisfied, be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours.
>
> b. With the ultimate heat sink (UHS) inoperable for any reason other than the temperature, be in at least hot standby within the 6 hours and in cold shutdown within the following 30 hours.

The new action statement only applies to the case of the plant inlet water temperature is above the TS limit of 90°F, requires more frequent checking of the temperature above 90°F and plant cooling systems, only allows operation above 90°F for the limited time period of 12 hours, and does not change the time period in the current action statement to shut down.

## 3.0 EVALUATION

The ultimate heat sink (UHS) for WCGS is the normally submerged Seismic Category I cooling pond. The UHS is formed by providing a volume of cooling water behind a Seismic Category I dam built in one finger of the WCGS cooling lake. The two principal functions of the UHS are the dissipation of residual heat after reactor shutdown, and dissipation of residual heat after an accident. The basic performance requirements for the UHS are that a 30-day supply of water be available, and that the design-basis temperatures of safety-related equipment not be

exceeded. The UHS design assures that the design temperature of safety-related equipment is not exceeded. The design temperature of water supplied to the plant is 95°F.

The UHS is the sink for heat removed from the reactor core following all accidents and anticipated operational occurrences in which the unit is cooled down and placed on residual heat removal (RHR) operation. Its maximum post-accident heat load occurs after a design basis loss-of-coolant-accident (LOCA) when the unit switches from injection to recirculation and the containment cooling system and RHR are required to remove the core decay heat. Section 9.2.5 of the Updated Safety Analysis Report (USAR) provides the details of the assumptions used in the heat transfer analysis for the worst-case LOCA, which include worst expected meteorological conditions, conservative uncertainties when calculating decay heat, and worst-case single failure. In addition, for the analysis, it was assumed that all of the water in the UHS was at 90°F at the start of the event.

The analysis predicted 95°F as the highest plant inlet water temperature occurring during the maximum temperature period following the loss of the main dam with the average water temperature generally being below 94°F.

The licensee evaluated the effect of the proposed change on normal plant operation and normal plant shutdown with the main dam intact, and safe shutdown or post-accident operation without the main dam.

Normal Plant Operation with the Main Dam Intact: Short-term operation with an inlet water temperature of up to 95°F is not expected to negatively affect plant operation, with the possible exception of turbine backpressure. A slight load reduction may be necessary to maintain acceptable turbine backpressure. Existing plant guidance will be employed if any unexpected transients are experienced.

Shutdown with the Main Dam Intact: Increasing the inlet water temperature from 90°F to 95°F may cause the calculated single train time required to cool the unit below 200°F to exceed the cooldown time needed to comply with some TS. To compensate for this concern, the requested action statement requires verification of operability of two RHR trains when the inlet water temperature is greater than 90°F. This will ensure the cooling capacity is available to meet the shutdown time requirements.

LOCA with the Main Dam Intact: The effect of full power plant operation on plant inlet water temperature during worst case predicted summer environmental conditions is approximately 0.5°F. The peak heat rejection rate by the plant post-LOCA would be approximately 5 percent of the continuous heat rejection rate of the plant during normal operation. Therefore, the effect of post-LOCA heat loads on plant inlet water temperature would be less than 0.1°F. The current UHS analysis assumes that there has been a main dam failure and uses worst case environmental conditions. The results indicate that with an initial UHS temperature of 90°F, plant intake water temperature will not exceed 95°F. The UHS analysis results also indicate that the environmental conditions have a much greater effect on peak plant intake water temperature than does the heat rejected from the plant. The current UHS analysis is recognized as bounding the LOCA condition without a main dam failure because the volume of the UHS is significantly smaller than the volume of the WCGS cooling lake, approximately 1

percent. The probability of environmental conditions significantly worse than those causing entry into the limiting condition for operation is low. The probability of these conditions occurring simultaneously with a LOCA is even lower.

Safe Shutdown or Post-Accident Operation without the Main Dam: The TS limit of 90°F is not being changed; however, the license amendment provides an allowance for operation above that limit for a 12-hour period. The Wolf Creek accident analyses assume a plant inlet water temperature of 95°F. Based on a review of recent WCGS cooling lake data, 12 hours may be necessary to restore the lake below 90°F through diurnal effects. Safe shutdown capability and post-accident operation without the main dam is ensured when the plant is operated within TS limits. The probability of main dam failure is low, comparable to the frequency of a large break LOCA initiating event. The probability of main dam failure during the 12 hours when the inlet water temperature is above 90°F in conjunction with an accident is even lower. A seismic event is a possible initiating event for causing failure of the main dam. The frequency of the seismic initiator on an annual basis is nearly equal to that of a large break LOCA. It is also noted that WCGS has a dam monitoring program in place to ensure continued integrity of the main dam. Therefore, it is concluded that this proposed change is of low risk significance.

Based on the above review and the license amendment No. 118 dated July 18, 1998, the staff concludes that with the dam intact, adequate heat removal will be available during normal plant operation, shutdown, and LOCA conditions to maintain equipment temperatures at or below their maximum design temperature of 95°F. Further, the staff concludes that the probability of a LOCA concurrent with a dam failure is very low and, therefore, acceptable.

Based on the above, the staff finds the proposed change to the WCGS TS to add a new action statement to support continued plant operation in the event that plant inlet water temperature exceeds 90°F and remains less than 95°F acceptable. Therefore, limiting the plant inlet water temperature limit to 94°F is also acceptable. The licensee's proposal to raise the plant inlet water temperature at 95°F will be addressed in a later evaluation.

## 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment on July 8, 1999. The State official had no objections to the amendment.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendment involves no significant hazards consideration, and there have been no public comment on such findings (64 FR 9203). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental

impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Thomas

Date: July 8, 1999