

March 30, 1998

Mr. Otto L. Maynard
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 115 TO FACILITY
OPERATING LICENSE NO. NPF-42 (TAC NO. MA0732)

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. 115 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated January 28, 1998.

The amendment revises TS Sections 6.3 and 6.12 to reflect a merger for the positions of Superintendent Radiation Protection and Superintendent Chemistry into one new position, Manager Chemistry/Radiation Protection.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Kristine M. Thomas

Kristine M. Thomas, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 115 to NPF-42
2. Safety Evaluation

cc w/encls: See next page

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Docket File	OGC, 015B18
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PDIV-2 Reading	GHill (2)
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LHurley, RIV	SBlack

DOCUMENT NAME: WCMA0732.AMD

OFFICE	PDIV-2/PM	PDIV-2/LA	HHFB HGMB/BC	OGC
NAME	<i>KMT</i> KThomas	<i>EEP</i> EPeyton	<i>DE [unclear]</i> SBlack	<i>WBeckner</i> WBeckner
DATE	2/11/98	2/10/98	2/12/98	2/24/98

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Mr. Otto L. Maynard

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March 30, 1998

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 115
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated January 28, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 115 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Kristine M. Thomas

Kristine M. Thomas, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 30, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 115

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

6-7
6-23

INSERT

6-7
6-23

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

- b. The position of Manager Chemistry/Radiation Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, for a Radiation Protection Manager.
- c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.
- d. The position of Manager Operations shall hold or have previously held a senior reactor operator license for a similar unit (PWR).

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

- a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.
- b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Manager shall designate in writing the Chairman and Alternate Chairman of the PSRC. PSRC membership shall include between six and eight additional members appointed by the Chairman and an additional member appointed by the Vice President Engineering. Selected members shall include, at a minimum, management responsible for the following areas of expertise: operations, maintenance, instrumentation and controls, chemistry, health physics and engineering. A single individual may cover multiple disciplines.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.**

**Except for the alternate for Engineering who is appointed by the Vice President Engineering.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) Administrative Control Procedures and changes thereto, and (2) procedures and changes thereto required by Specification 6.8.1 and requiring a 10 CFR 50.59 safety evaluation.
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant Manager, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Not used.
- j. Not used.

ADMINISTRATIVE CONTROLS

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by Paragraph 20.203(c)(2), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area, or
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them, or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Manager Chemistry/Radiation Protection in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Supervisor/Supervising Operator on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed-circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

ADMINISTRATIVE CONTROLS

HIGH RADIATION AREA (Continued)

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Plant Manager.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Plant Manager.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By letter dated January 28, 1998, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specification (TS) Sections 6.3 and 6.12 to reflect a position change within the Wolf Creek Nuclear Operating Corporation (WCNOC) organization. Specifically, the change would reflect a merger of the positions of Superintendent Radiation Protection and Superintendent Chemistry into one new position, Manager Chemistry/Radiation Protection.

2.0 EVALUATION

The duties of the Superintendent Radiation Protection will be merged with the duties of the Superintendent Chemistry into one new position, Manager Chemistry/Radiation Protection. The replacement of the two supervisory positions with one management position will not change any of the qualification requirements specified in the TS. The new position of Manager Chemistry/Radiation Protection will meet or exceed the qualifications of Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants," September 1975, for a Radiation Protection Manager. The new position will report to the Plant Manager, as the Superintendent Chemistry and Superintendent Radiation Protection currently do. This proposed change will not affect any other positions. The structure of and the functions performed by the two groups under the two supervisory positions, Chemistry Technicians and Radiation Protection Technicians, are very similar. Merging the two positions into one management position will enable WCNOC to increase communication effectiveness between the two groups, allowing both groups to work together more effectively. These changes are administrative in nature and will enhance the organizational structure of WCNOC. These changes will not reduce overall organizational commitments.

Based on the above, the staff finds the proposed change to merge the positions of Superintendent Radiation Protection and Superintendent Chemistry into one new position, Manager Chemistry/Radiation Protections, acceptable.

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3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Thomas

Date: March 30, 1998