

10 CFR 50.90

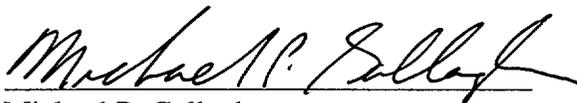
July 11, 2002
2130-02-20187U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001Subject: Response To Request For Additional Information –
Technical Specification Change Request No. 308
General Electric Stability Analysis Methodology (TAC No. MB4960)Oyster Creek Generating Station
Facility Operating License No. DPR-16
NRC Docket No. 50-219

This letter provides additional information in response to NRC request for additional information (RAI) dated June 17, 2002 and as discussed in a conference call on June 27, 2002 regarding Oyster Creek Technical Specification Change Request (TSCR) No. 308, submitted to NRC for review on April 26, 2002. As clarified in the conference call held on June 27, 2002, NRC Questions 1 through 8 of the June 17, 2002 RAI are unrelated to the scope of TSCR No. 308 and a docketed response is not required. The additional information regarding the remaining issue is enclosed.

No new regulatory commitments are established by this submittal. If any additional information is needed, please contact David J. Distel (610) 765-5517.

I declare under penalty of perjury that the foregoing is true and correct.

Very truly yours,

07-11-02
Executed On
Michael P. Gallagher
Director, Licensing & Regulatory Affairs
Mid Atlantic Regional Operating Group

Enclosure: Response to Request for Additional Information

c: H. J. Miller, USNRC Administrator, Region I
P. S. Tam, USNRC Senior Project Manager, Oyster Creek
R. J. Summers, USNRC Senior Resident Inspector, Oyster Creek
File No. 02033

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ENCLOSURE

Oyster Creek Generating Station

**Response to Request for Additional Information
Technical Specification Change Request No. 308
General Electric Stability Analysis Methodology**

Oyster Creek Generating Station
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NRC Question

If your in-house setpoint calculation methodology was previously approved by the NRC, please cite the approval document. If your methodology was not previously reviewed and approved by the NRC, please confirm that it is based on NRC-approved Industry standards, and meets the 95/95 confidence level requirement.

Response

Oyster Creek setpoint calculations are performed in accordance with AmerGen Engineering Standard ES-002. This methodology is based on ISA-67.04(1982) - Set Points for Nuclear Safety Related Instrumentation Used In Nuclear Power Plants. The methodology outlined in the AmerGen standard meets the 95/95 confidence level requirement.