

August 11, 1994

Docket No. 50-482

Mr. Neil S. Carns  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
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Dear Mr. Carns:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 75 TO FACILITY  
OPERATING LICENSE NO. NPF-42 (TAC NO. M89853)

The Commission has issued the enclosed Amendment No. 75 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated May 24, 1994. This amendment is a partial response to your application dated May 24, 1994. The remaining changes proposed in that application will be addressed by a separate amendment.

The amendment relocates the TS requirements related to seismic monitoring instrumentation from the TS to the Updated Safety Analysis Report (USAR). The requirements of these TS will be maintained and controlled pursuant to Appendix A to 10 CFR 100 and other applicable regulations, including 10 CFR 50.59, "Changes, tests, and experiments."

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

William D. Reckley, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Enclosures:

- Amendment No. 75 to NPF-42
- Safety Evaluation

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cc w/enclosures:

See next page

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Mr. Neil S. Carns

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August 11, 1994

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated May 24, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

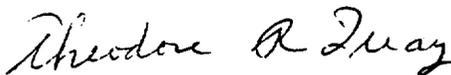
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 75, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 11, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

VI  
3/4 3-44  
3/4 3-45  
3/4 3-46

INSERT

VI  
3/4 3-44  
3/4 3-45  
3/4 3-46

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 AXIAL FLUX DIFFERENCE (AFD).....	3/4 2-1
3/4.2.2 HEAT FLUX HOT CHANNEL FACTOR - $F_{\theta}(X,Y,Z)$ .....	3/4 2-4
3/4.2.3 NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR - $F_{\Delta H}(X,Y)$ .....	3/4 2-9
3/4.2.4 QUADRANT POWER TILT RATIO.....	3/4 2-11
3/4.2.5 DNB PARAMETERS.....	3/4 2-14
TABLE 3.2-1 DNB PARAMETERS.....	3/4 2-16
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR TRIP SYSTEM INSTRUMENTATION.....	3/4 3-1
TABLE 3.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION.....	3/4 3-2
TABLE 3.3-2 REACTOR TRIP SYSTEM INSTRUMENTATION RESPONSE TIMES....	3/4 3-7
TABLE 4.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-9
3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-13
TABLE 3.3-3 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-14
TABLE 3.3-4 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS.....	3/4 3-22
TABLE 3.3-5 ENGINEERED SAFETY FEATURES RESPONSE TIMES.....	3/4 3-29
TABLE 4.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-34

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>INSTRUMENTATION</u> (Continued)	
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring for Plant Operations.....	3/4 3-39
TABLE 3.3-6 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS.....	3/4 3-40
TABLE 4.3-3 RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS SURVEILLANCE REQUIREMENTS.....	3/4 3-42
Movable Incore Detectors.....	3/4 3-43
Seismic Instrumentation.....	DELETED
TABLE 3.3-7 SEISMIC MONITORING INSTRUMENTATION.....	DELETED
TABLE 4.3-4 SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	DELETED
Meteorological Instrumentation.....	3/4 3-47
TABLE 3.3-8 METEOROLOGICAL MONITORING INSTRUMENTATION.....	3/4 3-48
TABLE 4.3-5 METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-49
Remote Shutdown Instrumentation.....	3/4 3-50
TABLE 3.3-9 REMOTE SHUTDOWN MONITORING INSTRUMENTATION.....	3/4 3-51
TABLE 4.3-6 REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-52
Accident Monitoring Instrumentation.....	3/4 3-53
TABLE 3.3-10 ACCIDENT MONITORING INSTRUMENTATION.....	3/4 3-54
TABLE 4.3-7 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-55
Chlorine Detection Systems.....	DELETED
Loose-Part Detection System.....	3/4 3-57
Radioactive Liquid Effluent Monitoring Instrumentation...	DELETED
TABLE 3.3-12 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION.....	DELETED

## INSTRUMENTATION

### MOVABLE INCORE DETECTORS

#### LIMITING CONDITION FOR OPERATION

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3.3.3.2 The Movable Incore Detection System shall be OPERABLE with:

- a. At least 75% of the detector thimbles,
- b. A minimum of two detector thimbles per core quadrant, and
- c. Sufficient movable detectors, drive, and readout equipment to map these thimbles.

APPLICABILITY: When the Movable Incore Detection System is used for:

- a. Recalibration of the Excore Neutron Flux Detection System,
- b. Monitoring the QUADRANT POWER TILT RATIO, or
- c. Measurement of  $F_Q(X,Y,Z)$  and  $F_{\Delta H}(X,Y)$ .

#### ACTION:

- a. With the Movable Incore Detection System inoperable, do not use the system for the above applicable monitoring or calibration functions.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.2 The Movable Incore Detection System shall be demonstrated OPERABLE at least once per 24 hours by normalizing each detector output when required for:

- a. Recalibration of the Excore Neutron Flux Detection System, or
- b. Monitoring the QUADRANT POWER TILT RATIO, or
- c. Measurement of  $F_Q(X,Y,Z)$  and  $F_{\Delta H}(X,Y)$ .

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. NPF-42  
WOLF CREEK NUCLEAR OPERATING CORPORATION  
WOLF CREEK GENERATING STATION  
DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated May 24, 1994, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes included a variety of revisions and relocations of technical specification (TS) requirements as part of the licensee's implementation of the NRC's Final Policy Statement on Technical Specifications Improvements (58 FR 39132). One of the proposed changes relocates the TS requirements related to seismic monitoring instrumentation from the TS to the Updated Safety Analysis Report (USAR). The relocated TS requirements will be maintained and controlled pursuant to Appendix A of 10 CFR 100 and other applicable regulations, including 10 CFR 50.59, "Changes, tests, and experiments."

2.0 EVALUATION

Section 50.36 of Title 10 of the *Code of Federal Regulations* established the regulatory requirements related to the content of TS. The rule requires that TS include items in specific categories, including safety limits, limiting conditions for operation, and surveillance requirements; however, the rule does not specify the particular requirements to be included in a plant's TS. The NRC developed criteria, as described in the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors," to determine which of the design conditions and associated surveillances need to be located in the TS "to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety." Briefly, those criteria are (1) detection of abnormal degradation of the reactor coolant pressure boundary, (2) boundary conditions for design-basis accidents and transients, (3) primary success paths to prevent or mitigate design-basis accidents and transients, and (4) functions determined to be important to risk or operating experience. The Commission's final policy statement acknowledged that its implementation may result in the relocation of existing technical specification requirements to licensee controlled documents and programs.

Appendix A to 10 CFR Part 100 requires that seismic monitoring instrumentation be provided to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to allow for a comparison of the measured response to that used in the design basis for the unit. Comparison of such data is needed to (1) determine whether the plant can continue to be operated safely and (2) permit such timely action as may be appropriate. However, these components are not factored into accident analyses nor do they affect the margin of safety of the plant. Seismic instrumentation does not actuate any protective equipment or play any direct role in the mitigation of an accident. The capability of the plant to withstand a seismic event or other design-basis accident is determined by the initial design and construction of systems, structures, and components. The instrumentation is used to alert operators to the seismic event and evaluate the plant response. Therefore, the requirements of TS 3/4.3.3 and associated TS Bases related to the seismic monitoring instrumentation do not satisfy any of the above final policy statement criteria and need not be included in the TS. In addition, the proposed amendment does not involve a change in the manner in which the plant will be operated, maintained, or tested. The requirements described in the affected TS will be maintained, and any subsequent changes to the plant procedures or the USAR related to these instruments will be made in accordance with 10 CFR 50.59.

On this basis, the staff concludes that the requirements related to seismic instrumentation do not need to be controlled by TS, and changes to the seismic instrumentation are adequately controlled by 10 CFR 50.59, "Changes, tests, and experiments." Should the licensee's determination conclude that an unreviewed safety question is involved, due to either (1) an increase in the probability or consequences of accidents or malfunctions of equipment important to safety, (2) the creation of a possibility for an accident or malfunction of a different type than any evaluated previously, or (3) a reduction in the margin of safety, NRC approval and a license amendment would be required prior to implementation of the change. NRC inspection and enforcement programs also enable the staff to monitor facility changes and licensee adherence to USAR commitments and to take any remedial action that may be appropriate.

The staff has concluded, therefore, that relocation of the requirements related to seismic monitoring instrumentation is acceptable because (1) their inclusion in technical specifications is not specifically required by 10 CFR 50.36 or other regulations, (2) these requirements are not required to avert an immediate threat to the public health and safety, and (3) changes that are deemed to involve an unreviewed safety question will require prior NRC approval in accordance with 10 CFR 50.59(c).

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. D. Reckley

Date: August 11, 1994