

August 1, 1996

Mr. Neil S. Carns  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 100 TO FACILITY  
OPERATING LICENSE NO. NPF-42 (TAC NO. M95323)

Dear Mr. Carns:

The Commission has issued the enclosed Amendment No. 100 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated May 1, 1996.

The amendment revises TS Section 6.0, throughout, to reflect an organization change in which the position of Vice President Plant Operations has been eliminated and the positions of Chief Operating Officer and Plant Manager were created. This change assigns certain management responsibilities to the Chief Operating Officer and Plant Manager.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By  
James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 100 to NPF-42  
2. Safety Evaluation

cc w/encls: See next page

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Docket File OGC, 015B18  
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PDIV-2 Reading GHill (2)  
EGA1 CGrimes, 011E22  
WJohnson, RIV JRoe  
JStone ACRS  
EPeyton CVM (SE)  
LHurley, RIV SBlack

DOCUMENT NAME: WC95323.AMD

OFC	LA:PDIV-2	PDIV-2 <i>JH</i>	HQMB*	OGC <i>OPW</i>	PDIV-2
NAME	<i>ESP</i> EPeyton	JStone	SBlack		WBateman
DATE	7/17/96	8/1/96	6/24/96	7/19/96	7/ /96

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 1, 1996

Mr. Neil S. Carns  
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Wolf Creek Nuclear Operating Corporation  
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SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 100 TO FACILITY  
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Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 100 to NPF-42  
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Mr. Neil S. Carns

- 2 -

August 1, 1996

cc w/encls:

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Shaw, Pittman, Potts & Trowbridge  
2300 N Street, NW  
Washington, D.C. 20037

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Senior Resident Inspector  
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Chief Engineer  
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Kansas Corporation Commission  
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State of Kansas  
Topeka, Kansas 66612

Attorney General  
Judicial Center  
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2nd Floor  
Topeka, Kansas 66612

County Clerk  
Coffey County Courthouse  
Burlington, Kansas 66839

Public Health Physicist  
Bureau of Air & Radiation  
Division of Environment  
Kansas Department of Health  
and Environment  
Forbes Field Building 283  
Topeka, Kansas 66620



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100  
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated May 1, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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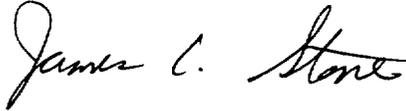
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

- 7 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 109 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 100

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

6-1  
6-7  
6-8  
6-9  
6-14  
6-24

INSERT

6-1  
6-7  
6-8  
6-9  
6-14  
6-24

## ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall Unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Supervising Operator, under the Shift Supervisor, shall be responsible for the control room command function. A management directive to this effect, signed by the President and Chief Executive Officer shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### 6.2.1 Onsite and Operating Corporation Organization

Onsite and operating corporation organizations shall be established for unit operation and corporate management, respectively. The onsite and operating corporation organizations shall include the positions for the activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or equivalent forms of documentation. These requirements shall be documented in the Updated Safety Analysis Report.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control of those onsite activities necessary for safe operation and maintenance of the plant.
- c. The President and Chief Executive Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out the health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

#### 6.2.2 Unit Staff

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

## ADMINISTRATIVE CONTROLS

### Unit Staff (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the Unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual from the Health Physics Group\*, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least 5 members\* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the Unit and any personnel required for other essential functions during a fire emergency; and
- f. Administrative procedures shall be developed and implemented to limit the working hours of Unit Staff who perform safety-related functions; e.g., Senior Operators, Operators, Health Physicists, Auxiliary operators, and key maintenance personnel.  
  
The amount of overtime worked by Unit Staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).
- g. The Superintendent Operations or Manager Operations shall hold a senior reactor operator license.

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\*May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

## ADMINISTRATIVE CONTROLS

### 6.3 UNIT STAFF QUALIFICATIONS (Continued)

- b. The position of Superintendent Radiation Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, for a Radiation Protection Manager.
- c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.
- d. The position of Manager Operations shall hold or have previously held a senior reactor operator license for a similar unit (PWR).

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

- a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.
- b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

##### FUNCTION

6.5.1.1 The PSRC shall function to advise the Plant Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The Plant Manager shall designate in writing the Chairman and Alternate Chairman of the PSRC. PSRC membership shall include between six and eight additional members appointed by the Chairman and an additional member appointed by the Vice President Engineering. Selected members shall include, at a minimum, management responsible for the following areas of expertise: operations, maintenance, instrumentation and controls, chemistry, health physics and engineering. A single individual may cover multiple disciplines.

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.\*\*

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\*\*Except for the alternate for Engineering who is appointed by the Vice President Engineering.

## ADMINISTRATIVE CONTROLS

### Unit Staff (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the Unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual from the Health Physics Group\*, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least 5 members\* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the Unit and any personnel required for other essential functions during a fire emergency; and
- f. Administrative procedures shall be developed and implemented to limit the working hours of Unit Staff who perform safety-related functions; e.g., Senior Operators, Operators, Health Physicists, Auxiliary operators, and key maintenance personnel.  
  
The amount of overtime worked by Unit Staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).
- g. The Superintendent Operations or Manager Operations shall hold a senior reactor operator license.

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\*May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

## ADMINISTRATIVE CONTROLS

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### MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

### QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) Administrative Control Procedures and changes thereto, and (2) procedures and changes thereto required by Specification 6.8.1 and requiring a 10 CFR 50.59 safety evaluation.
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant Manager, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Not used.
- j. Not used.

## ADMINISTRATIVE CONTROLS

### RESPONSIBILITIES (Continued)

- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL and the Radwaste Treatment Systems, and
- l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Plant Manager and to the Nuclear Safety Review Committee.
- m. Review of the Fire Protection Program and shall submit recommended changes to the Nuclear Safety Review Committee.

#### 6.5.1.7 The PSRC shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specification 6.5.1.6a. through d. and m. above,
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. through e. above constitutes an unreviewed safety question, and
- c. Provide written notification within 24 hours to the Chief Operating Officer and the Nuclear Safety Review Committee of disagreement between the PSRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

### RECORDS

6.5.1.8 The PSRC shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Plant Manager and the Nuclear Safety Review Committee.

### 6.5.2 NUCLEAR SAFETY REVIEW COMMITTEE (NSRC)

#### FUNCTION

6.5.2.1 The NSRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,

## ADMINISTRATIVE CONTROLS

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### FUNCTION (Continued)

- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The NSRC shall report to and advise the President and Chief Executive Officer on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

### COMPOSITION

6.5.2.2 The NSRC shall be composed of at least eight designated members, including the Chairman. Members of the NSRC may be from within the WCNOG organization or from outside organizations. The NSRC shall have sufficient expertise to adequately provide an independent review and audit of designated activities in the areas listed in Technical Specification Section 6.5.2.1. Additional members may be appointed by the Chairman.

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRC Chairman to provide expert advice to the NSRC.

### MEETING FREQUENCY

6.5.2.5 The NSRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

### QUORUM

6.5.2.6 The quorum of the NSRC necessary for the performance of the NSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least two-thirds of the NSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the Unit.

## ADMINISTRATIVE CONTROLS

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### MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

### QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) Administrative Control Procedures and changes thereto, and (2) procedures and changes thereto required by Specification 6.8.1 and requiring a 10 CFR 50.59 safety evaluation.
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Plant Manager, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Not used.
- j. Not used.

## ADMINISTRATIVE CONTROLS

### RECORDS (Continued)

- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the President and Chief Executive Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 of 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and submitted to the NSRC and the President and Chief Executive Officer.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The President and Chief Executive Officer and the NSRC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective ACTION taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the President and Chief Executive Officer within 14 days of the violation; and
- d. Critical operation of the Unit shall not be resumed until authorized by the Commission.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A, of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Section 7.1 of Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;

## ADMINISTRATIVE CONTROLS

### PROCEDURES AND PROGRAMS (Continued)

- e. Process Control Program implementation;
- f. ODCM implementation; and
- g. Quality Assurance Program implementation for effluent and environmental monitoring.
- h. Fire Protection Program implementation.

6.8.2 Each procedure of Specification 6.8.1 and changes thereto, and any other procedure or procedure change that the Plant Manager determines to affect nuclear safety, shall be reviewed and approved as described below, prior to implementation.

- a. Each procedure, or change thereto shall be reviewed by a Qualified Reviewer who is knowledgeable in the functional area affected, but is not the individual who prepared the procedure or procedure change. All required cross-disciplinary reviews of new procedures, procedure revisions or changes thereto shall be completed prior to approval.
- b. Procedures other than Administrative Control Procedures shall be approved by the responsible Manager or his designee as specified in Administrative Control Procedures. The Plant Manager shall approve Administrative Control Procedures. The Manager responsible for the Security Plan shall approve the Security Plan and implementing procedures. The Manager responsible for Emergency Planning shall approve the Radiological Emergency Response Plan and implementing procedures.
- c. The responsible Manager or his designee shall ensure each review includes a determination of whether a procedure, or change thereto, requires a 10 CFR 50.59 safety evaluation. If a procedure, or change thereto, requires a 10 CFR 50.59 safety evaluation, the responsible Manager or his designee shall forward the procedure, or change thereto with the associated 10 CFR 50.59 safety evaluation to the PSRC for review in accordance with Specification 6.5.1.6.a. Pursuant to Section 50.59, 10 CFR, NRC approval of items involving unreviewed safety questions shall be obtained prior to approval for implementation.
- d. Qualified Reviewers shall meet the applicable qualifications of ANSI/ANS 3.1-1978. Personnel recommended to be Qualified Reviewers shall be reviewed by the PSRC and approved and documented by the PSRC Chairman. The responsible Manager shall ensure that a sufficient complement of Qualified Reviewers for their functional area is maintained in accordance with Administrative Control Procedures.
- e. Records documenting the activities performed under Specification 6.8.2.a. through 6.8.2.d, shall be maintained in accordance with Specification 6.10.

## ADMINISTRATIVE CONTROLS

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by Paragraph 20.203(c)(2), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area, or
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them, or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Superintendent Radiation Protection in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Supervisor/Supervising Operator on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed-circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

## ADMINISTRATIVE CONTROLS

### HIGH RADIATION AREA (Continued)

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

### 6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
  - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
  - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Plant Manager.

### 6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
  - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
  - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Plant Manager.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By letter dated May 1, 1996, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specification (TS) Section 6.0 to reflect an organization change in which the position of Vice President Plant Operations has been eliminated and the positions of Chief Operating Officer and Plant Manager have been created. The duties currently assigned to the Vice President Plant Operations have been reassigned to the Plant Manager.

2.0 BACKGROUND

Wolf Creek has proposed an organizational change that eliminated the position of the Vice President Operations and created the positions of Chief Operating Officer and Plant Manager. This change will designate the responsibilities of each of these positions.

Specifically, the duties of Vice President Plant Operations have been reassigned to the Plant Manager in Sections 6.1.1, 6.2.1.b, 6.5.1.1, 6.5.1.2, 6.5.1.6, 6.5.1.7, 6.5.1.8, 6.8.2, 6.13, and 6.14 of the technical specifications.

Section 6.5.1.7.c has been revised to indicate that in the event of disagreement between the Plant Safety Review Committee (PSRC) and the Plant Manager, that the Chief Operating Officer, rather than the Chief Executive Officer, shall be provided written notification within 24 hours.

3.0 EVALUATION

The following duties, which were previously assigned to the Vice President Plant Operations, have been assigned to the Plant Manager:

Overall operation of the Unit (6.1.1)

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Overall operation of the Unit and control of onsite activities necessary for safe operation and maintenance (6.2.1.b).  
The Plant Safety Review Committee (PSRC) shall advise the Plant Manager on all matters related to nuclear safety (6.5.1.1).

Designate the Chairman and Vice Chairman of the PSRC (6.5.1.2).

Receive all reports of technical specification violations from the PSRC (6.5.1.6.e).

Receive all reports of accidental, unplanned, or uncontrolled radioactive releases from the PSRC (6.5.1.6.1).

Receive recommendations of approval or disapproval, from the PSRC, of items reviewed by the PSRC (6.5.1.7.a).

When disagreements arise between the PSRC and the Plant Manager, the Plant Manager is ultimately responsible for resolution of the disagreement (6.5.1.7.c).

Receive copies of the PSRC minutes (6.5.1.8).

Determines which procedures and changes that affect nuclear safety are reviewed and approved in accordance with the technical specification requirements (6.8.2).

Approve administrative control procedures (6.8.2.b).

Approve changes to process control procedures (6.13.b).

Approve the offsite dose calculation manual (6.14.b).

The creation of the position of Plant Manager did not change any of the qualification requirements in the technical specifications and the licensee did not request any changes to the qualification requirements of the position. The positions below the Plant Manager have not changed.

Section 6.5.1.7.c of the technical specifications currently requires the PSRC to notify the President and Chief Operating Officer and the Nuclear Safety Review Committee (NSRC) when a disagreement between the PSRC and the Plant Manager occurs. This would be changed to require the Chief Operating Officer and the NSRC be notified of the disagreement.

The Unit Staff will continue to meet the qualification of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Rev. 2, "Qualification and Training of Personnel for Nuclear Power Plants," as required by Section 6.3.1 of the technical specifications. Therefore, the duties assigned in the technical specifications are being carried out at an appropriate level within the Wolf Creek organization. Also, the qualification requirements, as specified in the technical specifications, have not been changed.

This reorganization provides increased emphasis on plant operation. This will allow increased direct supervision of the day-to-day operation of the plant by the Plant Manager. The other duties that are not directly associated with the day-to-day operation of the plant, i.e., training, emergency planning, scheduling, security, etc., have been assigned to the Chief Operating Officer. These changes allow the Plant Manager to focus on those issues that directly affect the operation of the plant. Based on the above discussion, the staff finds these changes acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Stone

Date: August 1, 1996