November 28, 1994

Mr. Neil S. Carns
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. M89968)

Dear Mr. Carns:

The Commission has issued the enclosed Amendment No. 80 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated July 15, 1994.

The amendment revises Technical Specification Table 4.3-3, Radiation Monitoring Instrumentation for Plant Operations Surveillance Requirements, to change the analog channel operational test (ACOT) interval for selected radiation monitors from monthly to quarterly.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

James C. Stone, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 80 to NPF-42
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated July 15, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 80, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James C. Stone

James C. Stone, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 28, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains marginal lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE	INSERT
3/4 3-42	3/4 3-42

TABLE 3.3-6 (Continued)

TABLE NOTATIONS

*With fuel in the respective fuel storage pool.

**With irradiated fuel in the fuel storage areas or fuel building.

#Trip Setpoint concentration value ($\mu Ci/cm^3$) is to be established such that the actual submersion dose rate would not exceed 2 mR/h in the control room.

##Trip Setpoint concentration value ($\mu Ci/cm^3$) is to be established such that the actual submersion dose rate would not exceed 4 mR/h in the fuel building.

###Trip Setpoint concentration value ($\mu Ci/cm^3$) is to be established such that the actual submersion dose rate would not exceed 9 mR/h in the containment building. The Setpoint value may be increased up to the equivalent limits of Section 3.1 of the ODCM in accordance with the methodology and parameters in the ODCM during containment purge or vent provided the Setpoint value does not exceed twice the maximum concentration activity in the containment determined by the sample analysis performed prior to each release in accordance with Table 3-1 of the ODCM.

ACTION STATEMENTS

ACTION 26 - With less than the Minimum Channels OPERABLE requirement, operation may continue provided the containment purge valves are maintained closed.

ACTION 27 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, isolate the Control Room Emergency Ventilation System and initiate operation of the Control Room Emergency Ventilation System in the recirculation mode within 72 hours, or with no OPERABLE channels within 1 hour.

ACTION 28 - With less than the Minimum Channels OPERABLE requirement, operation may continue for up to 30 days provided an appropriate portable continuous monitor with the same Alarm Setpoint is provided in the fuel area. Restore the inoperable monitors to OPERABLE status within 30 days or suspend all operations involving fuel movement in the fuel building.

ACTION 29 - Must satisfy the ACTION requirements for Specification 3.4.6.1.

ACTION 30 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, isolate the Fuel Building Ventilation System and initiate operation of the Emergency Exhaust System to maintain the fuel building at a negative pressure within 72 hours, or with no OPERABLE channels within 1 hour.

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	ANALOG CHANNEL OPERATIONAL TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
1. Containment				
a. Containment Atmosphere-Gaseous Radioactivity-High (GT-RE-31 & 32)	S	R	Q	All
b. Gaseous Radioactivity-RCS Leakage Detection (GT-RE-31 & 32)	S	R	Q	1, 2, 3, 4
c. Particulate Radioactivity - RCS Leakage Detection (GT-RE-31 & 32)	S	R	Q	1, 2, 3, 4
2. Fuel Building				
a. Fuel Building Exhaust-Gaseous Radioactivity-High (GG-RE-27 & 28)	S	R	Q	**
b. Criticality-High Radiation Level				
1) Spent Fuel Pool (SD-RE-37 & 38)	S	R	Q	*
2) New Fuel Pool (SD-RE-35 & 36)	S	R	Q	*
3. Control Room Air Intake-Gaseous Radioactivity-High (GK-RE-04 & 05)	S	R	Q	All

*With fuel in the respective fuel storage pool.
**With irradiated fuel in the fuel storage areas or fuel building.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated July 15, 1994, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specification Table 4.3-3, Radiation Monitoring Instrumentation for Plant Operations Surveillance Requirements, to increase the analog channel operational test (ACOT) interval for the following radiation monitors from monthly to quarterly:

- Containment Atmosphere - Gaseous Radioactivity - High (GT-RE-31 and 32)
- Gaseous Radioactivity - RCS Leakage Detection (GT-RE-31 and 32)
- Particulate Radioactivity - RCS Leakage Detection (GT-RE-31 and 32)
- Fuel Building Exhaust - Gaseous Radioactivity - High (GG-RE-27 and 28)
- Criticality - High Radiation Level (SD-RE-37 and 38, SD-RE-35 and 36)
- Control Room Air Intake - Gaseous Radioactivity - High (GK-RE-04 and 05)

This proposed change is identified as a line item improvement in Generic Letter 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation," and recommends the ACOT interval be changed from monthly to quarterly.

2.0 BACKGROUND

The following safety-related radiation monitors: control room air supply (GK-RE-04 and 05), fuel building exhaust (GG-RE-27 and 28), and containment atmosphere (GT-RE-31 and 32), are designed to isolate the control room, fuel building and containment, respectively, if airborne radioactivity exceeds allowable limits. In addition, containment monitors GT-RE-31 and 32 provide for detection of gaseous and particulate leakage from the reactor coolant system. These monitors are redundant for single-failure protection, designed to meet seismic Category I requirements and are powered from a Class 1E power source.

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Monitors SD-RE-35, 36, 37, and 38, are part of the fuel building area radiation monitoring system and serve as the criticality monitors for the new spent fuel pool (SD-RE-35 and 36) and the spent fuel storage pool (SD-RE-37 and 38). These monitors are not safety-related and are powered from a non-Class 1E power source.

3.0 EVALUATION

The licensee has stated in their application that performing the ACOT on the radiation monitors requires access to the delicate instruments inside the radiation monitor cabinets. The monthly tests have the potential to cause damage to these delicate instruments. In addition, the licensee has reviewed ACOT data and determined that for the safety-related radiation monitors, no calibration or set-point problems have been experienced since the beginning of plant operation. Increasing the ACOT interval from monthly to quarterly will reduce the potential for instrument damage and reduce the instrument out-of-service time, thus effectively increasing system reliability and availability.

The criticality monitors have no function related to the safe shutdown of the plant or the capability to mitigate the consequences of an accident. Credit is not taken in the accident analysis for these monitors. Extending the ACOT interval will not adversely affect the operation of the instruments and will reduce the potential for instrument damage and reduce the instrument out-of-service time, thus effectively increasing system reliability and availability.

This is consistent with Generic Letter 93-05, Section 5.14. The staff finds the above changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 53845). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Stone

Date: November 28, 1994