

April 3, 1995

Mr. Neil S. Carns
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Carns:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 85 TO FACILITY
OPERATING LICENSE NO. NPF-42 (TAC NO. M91118)

The Commission has issued the enclosed Amendment No. 85 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated December 12, 1994.

The amendment clarifies the surveillance requirements for verifying the correct required position for the valves in the auxiliary feedwater system.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

James C. Stone, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 85 to NPF-42
2. Safety Evaluation

cc w/encls: See next page

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CMcCracken

DOCUMENT NAME: WC91118.AMD

OFC	PDIV-2/LA	PDIV-2/PM	NRR: SBLB	OGC
NAME	EPeyton	JStone:pk	CMcCracken #40	ETBOLCOR
DATE	3/20/95	3/21/95	3/21/95	3/23/95

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OFC	PDIV-2/LA	PDIV-2/PM	NRR:SDLB	OGC
NAME	EPeyton	JStone:pk	CMcCracken #40	EHOLCOM
DATE	3/29/95	3/21/95	3/21/95	3/23/95

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 3, 1995

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President and Chief Executive Officer
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Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 85 to NPF-42
2. Safety Evaluation

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Mr. Neil S. Carns

- 2 -

April 3, 1995

cc:

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State of Kansas
Topeka, Kansas 66612

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Topeka, Kansas 66612

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Coffey County Courthouse
Burlington, Kansas 66839

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Bureau of Air & Radiation
Division of Environment
Kansas Department of Health
and Environment
Forbes Field Building 283
Topeka, Kansas 66620



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated December 12, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

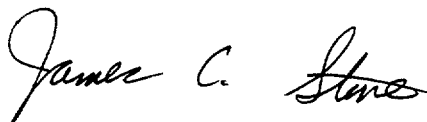
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

- A. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 85, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James C. Stone, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 3, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

3/4 7-5

INSERT

3/4 7-5

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 31 days by:

- 1) Verifying that each valve, except the auxiliary feedwater pump discharge control valves, in the flow path that is not locked, sealed, or otherwise secured in position is in its correct position; and
- 2) Verifying that each auxiliary feedwater pump discharge control valve in the flow path is in the fully open position whenever the Auxiliary Feedwater System is placed in standby or when above 10% RATED THERMAL POWER.

c. At least once per 18 months by:

- 1) Verifying that each automatic valve in the ESW supply to the auxiliary feedwater pumps actuates to its full open position upon receipt of an Auxiliary Feedwater Pump Suction Pressure-Low test signal;
- 2) Verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of an Auxiliary Feedwater Actuation test signal. For the steam turbine-driven auxiliary feedwater pump, the provisions of Specification 4.0.4 are not applicable for entry into MODE 3, and
- 3) Verifying that each auxiliary feedwater motor-operated discharge valve limits the flow to each steam generator from the motor-driven pump to less than or equal to 320 gpm.

4.7.1.2.2 An auxiliary feedwater flow path shall be demonstrated OPERABLE following each COLD SHUTDOWN of greater than 30 days prior to entering MODE 2 by verifying normal flow to at least two steam generators from one auxiliary feedwater pump.

PLANT SYSTEMS

CONDENSATE STORAGE TANK

LIMITING CONDITION FOR OPERATION

3.7.1.3 The condensate storage tank (CST) shall be OPERABLE with a contained water volume of at least 281,000 gallons.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

With the CST inoperable, within 4 hours either:

- a. Restore the CST to OPERABLE status or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours, or
- b. Demonstrate the OPERABILITY of the Essential Service Water (ESW) System as a backup supply to the auxiliary feedwater pumps and restore the CST to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.3.1 The CST shall be demonstrated OPERABLE at least once per 12 hours by verifying the contained water volume is within its limits when the tank is the supply source for the auxiliary feedwater pumps.

4.7.1.3.2 The ESW System shall be demonstrated OPERABLE at least once per 12 hours by verifying that the ESW System is in operation whenever the ESW System is the supply source for the auxiliary feedwater pumps.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated December 12, 1994, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specifications (TS) 4.7.1.2.1.b.1 and 4.7.1.2.1.b.2 to clarify the surveillance requirements for verifying the correct required position for the valves in the auxiliary feedwater (AFW) system.

2.0 BACKGROUND

Currently, Surveillance Requirement (SR) 4.7.1.2.1.b.1 requires verifying that each nonautomatic valve in the AFW system flow path that is not locked, sealed, or otherwise secured in position is in its correct position. Also, SR 4.7.1.2.1.b.2 requires verifying that each automatic valve in the AFW system flowpath is in the fully open position whenever the AFW system is placed in automatic control or when the reactor is above 10 percent rated thermal power. These surveillances are performed every 31 days in Modes 1, 2, or 3.

The licensee has proposed to change the wording of SR 4.7.1.2.1.b.1 to require verification that each valve, except the AFW pump discharge control valve, that is not locked, sealed, or otherwise secured in position is in its correct position. This surveillance requirement would be performed every 31 days. Also, a proposal to change SR 4.7.1.2.1.b.2 is included to require verification, every 31 days, that the AFW pump discharge control valves are in the fully open position whenever the AFW system is placed in standby or when the reactor is above 10 percent rated thermal power.

3.0 EVALUATION

Each motor-driven AFW pump discharges through a check valve and a locked-open isolation valve to feed two steam generators through individual sets of a locked open isolation valve, a normally open, motor-operated control valve, a check valve followed by a flow restriction orifice, and a locked-open globe valve. The turbine-driven AFW pump discharges through a check valve, a locked-open gate valve to each of the four steam generators through individual

sets of a locked-open isolation valve, a normally open air-operated control valve, followed by a check valve, a flow restriction orifice, and a locked-open globe valve.

When the AFW system is activated, the normally open control valves of the motor-driven AFW pumps are automatically operated to limit runout flow under all secondary side pressure conditions. This is required to prevent pump suction cavitation at high flow rates. The turbine-driven AFW pump design has a lower net positive suction head requirement. Because it is not as critical to reduce the flow from the turbine-driven AFW pump, the normally open discharge control valves are remote-manually operated.

The licensee currently administratively controls the turbine-driven AFW pump remote manual discharge control valves as automatically operated valves, although the Updated Safety Analysis Report states that these valves are remote-manually operated. This allows all of the AFW pump discharge control valves to be treated the same under SR 4.7.1.2.1.b.2. This change clarifies that all pump discharge valves are to be fully open whenever the AFW system is placed in standby or reactor power is above 10 percent. All other valves, including those on the suction side of the pumps would be verified to be in their correct position as required by SR 4.7.1.2.1.b.1.

The licensee also proposes that "automatic control" be changed to "standby" in TS 4.7.1.2.1.b.2. The licensee has stated that the Operations department at Wolf Creek does not utilize the terminology "automatic control," but instead uses "standby." This proposed change would not alter when the AFW pump discharge control valves are required to be in the fully open position from the current TS requirements.

The above changes clarify the position of the AFW pump discharge valves and change terminology to correspond with that used at Wolf Creek. These changes do not modify any TS associated with the AFW system. The staff finds these changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 3677). Accordingly, the amendment meets the eligibility criteria for

categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: James Stone, NRR

Date: April 3, 1995