

October 4, 1995

Mr. Neil S. Carns  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, Kansas 66839

Dear Mr. Carns:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 90 TO FACILITY  
OPERATING LICENSE NO. NPF-42 (TAC NO. M93002)

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated July 25, 1995.

The amendment deletes a clause from TS 4.0.5a, "Surveillance Requirements for Inservice Inspection and Testing Program." This clause required prior NRC approval before implementation of a relief request upon finding an ASME Code requirement impractical because of prohibitive dose rates or limitations in the design, construction, or system configuration.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 90 to NPF-42  
2. Safety Evaluation

cc w/encls: See next page

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DOCUMENT NAME: WC90032.AMD

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NAME	<i>esp</i> EPeyton	<i>zab</i> EABrown.pk	<i>JS</i> JStone	<i>RW</i> RWessman	<i>EH</i> EHoller
DATE	9/12/95	9/18/95	9/18/95	9/18/95	9/26/95

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Division of Reactor Projects III/IV  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "James C. Stone".

James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-482

Enclosures: 1. Amendment No. 90 to NPF-42  
2. Safety Evaluation

cc w/encls: See next page

Mr. Neil S. Carns

- 2 -

October 4, 1995

cc w/encs:

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Mr. Neil S. Carns  
President and Chief Executive Officer  
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Post Office Box 411  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90  
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated July 25, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 90, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James C. Stone, Senior Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: October 4, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

3/4 0-2

INSERT

3/4 0-2

### 3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

#### 3/4.0 APPLICABILITY

#### LIMITING CONDITION FOR OPERATION

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3.0.1 Compliance with the Limiting Conditions for Operation contained in the succeeding specifications is required during the OPERATIONAL MODES or other conditions specified therein; except that upon failure to meet the Limiting Conditions for Operation, the associated ACTION requirements shall be met.

3.0.2 Noncompliance with a specification shall exist when the requirements of the Limiting Condition for Operation and associated ACTION requirements are not met within the specified time intervals. If the Limiting Condition for Operation is restored prior to expiration of the specified time intervals, completion of the ACTION requirements is not required.

3.0.3 When a Limiting Condition for Operation is not met, except as provided in the associated ACTION requirements, within 1 hour action shall be initiated to place the unit in a MODE in which the specification does not apply by placing it, as applicable, in:

- a. At least HOT STANDBY within the next 6 hours,
- b. At least HOT SHUTDOWN within the following 6 hours, and
- c. At least COLD SHUTDOWN within the subsequent 24 hours.

Where corrective measures are completed that permit operation under the ACTION requirements, the action may be taken in accordance with the specified time limits as measured from the time of failure to meet the Limiting Condition for Operation. Exceptions to these requirements are stated in the individual specifications.

This Specification is not applicable in MODE 5 or 6.

3.0.4 Entry into an OPERATIONAL MODE or other specified condition shall not be made unless the conditions for the Limiting Condition for Operation are met without reliance on provisions contained in the ACTION requirements. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with ACTION requirements. Exceptions to these requirements are stated in the individual specifications.



## APPLICABILITY

### SURVEILLANCE REQUIREMENTS

---

4.0.1 Surveillance Requirements shall be met during the OPERATIONAL MODES or other conditions specified for individual Limiting Conditions for Operation unless otherwise stated in an individual Surveillance Requirement.

4.0.2 Each Surveillance Requirement shall be performed within the specified surveillance interval with a maximum allowable extension not to exceed 25% of the specified interval.

4.0.3 Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.2, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. The allowable outage time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

4.0.4 Entry into an OPERATIONAL MODE or other specified condition shall not be made unless the Surveillance Requirement(s) associated with a Limiting Condition of Operation has been performed within the stated surveillance interval or as otherwise specified. This provision shall not prevent passage through or to OPERATIONAL MODES as required to comply with ACTION requirements.

4.0.5 Surveillance Requirements for inservice inspection and testing of ASME Code Class 1, 2, and 3 components shall be applicable as follows:

- a. Inservice inspection of ASME Code Class 1, 2, and 3 components and inservice testing of ASME Code Class 1, 2, and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR Part 50, Section 50.55a.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By letter dated July 25, 1995, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise TS 4.0.5a, "Surveillance Requirements for Inservice Inspection and Testing Program," and Bases Section 3/4.4.10, "Structural Integrity." The TS changes relate to inservice inspection (ISI) and inservice testing (IST) requirements which are specified in Section 50.55a, "Codes and Standards," of Title 10 of the Code of Federal Regulations. The American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (the Code) is incorporated by reference as the requirements for ISI and IST (as specified in Section XI of the Code). The proposed change deletes a clause in TS Surveillance Requirement (SR) 4.0.5a in accordance with the recommendations of NUREG-1482, "Guidelines for Inservice Testing at Nuclear Power Plants." Bases Section 3/4.4.10 was relocated to the updated Safety Analysis Report (USAR) by Amendment No. 89 to Facility Operating License No. NPF-42 concerning the technical specification improvement program. Therefore, changes to that section will be done using the licensee's USAR change procedure as allowed by 10 CFR 50.59, "Changes, tests and experiments".

2.0 BACKGROUND

The Commission's final policy statement on technical specifications improvements defines the scope of the technical specifications and provides a criterion for technical design items to be included in, or relocated out of the TS document. On July 19, 1995 (60 FR 36953), the NRC published the final rule governing the implementation of this policy via a revision of 10 CFR 50.36, "Technical Specifications," which became effective August 18, 1995. The April 7, 1995, revised version of the Standard Technical Specifications (STS) (NUREG-1431, Rev. 1), relocates the IST requirements to the administrative controls section of the TS and deletes a portion of the ISI requirements, retaining the reactor coolant pump (RCP) flywheel inspections in the administrative control section. NUREG-1482, Chapter 6, recommends that licensees revise their TSs to incorporate the revised STS for IST programs.

The 10-year interval for the Wolf Creek IST program began September 4, 1995, and the current ISI program is based on the requirements of the 1989 Edition of the ASME Code. The TS change will allow the licensee a period of 12 months from the beginning of the interval to identify, submit, and obtain approval of relief requests for impractical code requirements in accordance with 10 CFR 50.55a, paragraphs (f)(5) and (g)(5), for IST and ISI respectively.

### 3.0 EVALUATION

The licensee has made a revision to SR 4.0.5a deleting the clause requiring written relief from the Commission under all ISI and IST testing deviations. Wolf Creek Nuclear Operating Corporation based these revisions on the guidance of the draft NUREG-1482; however, subsequent revisions have incorporated guidance regarding relief from the Commission. If an impracticality is determined within the initial interval or within the first 12 months of a new interval, the licensee follows the requirements in 10 CFR 50.55a(f)(5)(iii) and (iv) or (g)(5)(iii) and (iv). If an impractical requirement is identified during subsequent intervals and not within the first 12 months, the licensee must meet the requirements of 10 CFR 50.55a(f)(5)(iii) or (g)(5)(iii), notify the Commission, submit the information supporting the determination of impracticality, and obtain NRC's approval pursuant to (f)(6)(i) or (g)(6)(i), prior to the time that the next test or inspection is required. However, the specification does not allow the licensee to implement alternative testing under paragraphs 50.55a (a)(3)(i) and (ii) until authorized by the Director of the Office of Nuclear Reactor Regulation.

These changes to the licensee's TS are consistent with the intent of the revised STS and the regulatory guidance in NUREG-1482. The ISI and IST requirements are given in 10 CFR 50.55a, which the licensee documents via its 10-year interval program requirements. The change is acceptable since the regulatory requirements are delineated in 10 CFR 50.55a, and the change eliminates inconsistencies between the TS and the regulations.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State Official was notified of the proposed issuance of the amendment. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 45191). Accordingly, the amendment meets the eligibility criteria for

categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: E. A. Brown

Date: October 4, 1995