December 24, 1991

Docket No. 50-482

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Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. M79886)

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated February 27, 1991, as supplemented by letter dated September 11, 1991.

The amendment revises Section 6.0 of the Technical Specifications to reflect miscellaneous changes to the administrative controls. Modifications include title changes in plant personnel, updated references, and clarifications regarding individuals responsible for assuming control room command and control.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

William D. Reckley, Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 54 to NPF-42
2. Safety Evaluation

cc w/enclosures:
See next page

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OFC	PDIV-2/LA	PDIV-2/PM	LHE:B/C	OGC	PDIV-2/D		
NAME	EPeyton	WReckley	JWermiel		SBlack		
DATE	12/17/91	12/17/91	12/12/91	12/17/91	12/19/91		

OFFICIAL RECORD COPY
Document Name: WC AMENDMENT/TAC NO. 79886

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated February 27, 1991, as supplemented by letter dated September 11, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 54, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Suzanne C. Black

Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 24, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
6-2	6-2
6-5	6-5
6-6	6-6
6-7	6-7

6.1 RESPONSIBILITY

6.1.1 The Director Plant Operations shall be responsible for overall Unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Supervising Operator, under the Shift Supervisor, shall be responsible for the control room command function. A management directive to this effect, signed by the President and Chief Executive Officer shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 Onsite and Operating Corporation Organization

Onsite and operating corporation organizations shall be established for unit operation and corporate management, respectively. The onsite and operating corporation organizations shall include the positions for the activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or equivalent forms of documentation. These requirements shall be documented in the Updated Safety Analysis Report.

b. The Director Plant Operations shall be responsible for overall unit safe operation and shall have control of those onsite activities necessary for safe operation and maintenance of the plant.

c. The President and Chief Executive Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

d. The individuals who train the operating staff and those who carry out the health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 Unit Staff

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

Unit Staff (Continued)

b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the Unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;

c. An individual from the Health Physics Group*, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;

d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;

e. A site Fire Brigade of at least 5 members* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the Unit and any personnel required for other essential functions during a fire emergency; and

f. Administrative procedures shall be developed and implemented to limit the working hours of Unit Staff who perform safety-related functions; e.g., Senior Operators, Operators, Health Physicists, Auxiliary operators, and key maintenance personnel.

The amount of overtime worked by Unit Staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

g. The Supervisor Operations or Manager Operations shall hold a senior reactor operator license.

*May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3, or 4	MODE 5 or 6
SS	1	1*
SO	1	None
RO	2	1
NSO	4	1
STA	1**	None
CHM	1	None

SS - Shift Supervisor with a Senior Operator license on Unit 1
SO - Supervising Operator with a Senior Operator license on Unit 1
RO - Individual with an Operator license on Unit 1
NSO - Nuclear Station Operator
STA - Shift Technical Advisor
CHM - Chemistry Personnel

The Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Supervising Operator from the control room while the unit is in MODE 1, 2, 3, or 4, an individual with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Supervising Operator from the control room while the Unit is in MODE 5 or 6, an individual with a valid Operator license shall be designated to assume the control room command function.

*One individual with a Senior Operator license, either Shift Supervisor or Supervising Operator.

**The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

6.2.3 INDEPENDENT SAFETY ENGINEERING GROUP (ISEG)

FUNCTION

6.2.3.1 The ISEG shall function to examine plant operating characteristics, NRC issuances, industry advisories, REPORTABLE EVENTS and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety. The ISEG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving plant safety to the Chairman Nuclear Safety Review Committee.

COMPOSITION

6.2.3.2 The ISEG shall be composed of at least five, dedicated, full-time engineers located on site. Each shall have a bachelor's degree in engineering or related science and at least 2 years professional level experience in his field.

RESPONSIBILITIES

6.2.3.3 The ISEG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

RECORDS

6.2.3.4 Records of activities performed by the ISEG shall be prepared, maintained, and forwarded each calendar month to Chairman Nuclear Safety Review Committee.

6.2.4 SHIFT TECHNICAL ADVISOR

The Shift Technical Advisor (STA)** shall provide technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering and plant analysis with regard to the safe operation of the Unit.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 with the following exceptions:

a. Licensed Operators and Senior Operators shall meet or exceed the qualifications of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2.

*Not responsible for sign-off function.

**The STA position shall be manned in MODES 1, 2, 3, and 4 unless the Shift Supervisor or the individual with a Senior Operator license meets the qualifications for the STA as required by the NRC.

6.3 UNIT STAFF QUALIFICATIONS (Continued)

b. The position of Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.

b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Director Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PSRC shall be composed of the:

Member:	Manager Nuclear Plant Engineering Wolf Creek
Member:	Manager Operations
Member:	Manager Technical Support
Member:	Manager Maintenance and Modifications
Member:	Manager Instrumentation and Control
Member:	Supervisor Reactor Engineering
Member:	Manager Radiation Protection
Member:	Manager Chemistry
Member:	Supervisor Results Engineering
Chairman:	Manager Plant Support

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

a. Review of: (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Director Plant Operations to affect nuclear safety;

b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;

c. Review of all proposed changes to Technical Specifications or the Operating License;

d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Operations, and to the Nuclear Safety Review Committee (NSRC);

f. Review of all REPORTABLE EVENTS;

g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;

i. Review of the plant Security Plan and implementing procedures and shall submit recommended changes to the NSRC;

j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the NSRC;

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated February 27, 1991, as supplemented by letter dated September 11, 1991, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Section 6.0 to reflect miscellaneous changes to the administrative controls at the Wolf Creek Generating Station. The September 11, 1991, letter provided clarifying information and minor editorial changes to the proposed Technical Specification revisions that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The licensee has made a number of proposed changes to the administrative section of the Technical Specifications. The changes will be discussed separately.

- The title of the Operations Supervisor found in Section 6.2.2.g is being changed to Supervisor Operations. This is being done to maintain consistency with the individual's actual title. Since this change is administrative in nature, the staff finds this proposed change acceptable.

- Section 6.2.2.g currently requires the Supervisor Operations (formerly Operations Supervisor as discussed above) to hold a senior reactor operator (SRO) license. The licensee proposes to expand this to permit either the Supervisor Operations or the Manager Operations to hold an SRO license. The stated purpose of this change is to provide organizational flexibility during the licensed operator requalification examination process. Since the proposed change will continue to provide a licensed SRO for supervision of the operating shift crews, the staff finds this change acceptable.

- Section 6.1.2 states that the Supervising Operator, under the Shift Supervisor, shall be responsible for the control room command function. Table 6.2-1, Minimum Shift Crew Composition, currently makes an improper reference to the Shift Supervisor (SS) as having control room

command function. The licensee has proposed correcting this condition by replacing references to the SS under Table 6.2-1 with the Supervising Operator. Since the proposed change corrects an improper reference and does not change the lines of authority, the staff finds this change acceptable.

- At the Wolf Creek facility, the SS typically assumes the dual role of SRO/Shift Technical Advisor. In addition, the designated SRO of Table 6.2-1, also serves as the Supervising Operator. This leaves the SS and Supervising Operator as the minimum SROs on shift. Table 6.2-1 states that when the Supervising Operator (formerly SS as discussed above) leaves the control room, "an individual (other than the Shift Technical Advisor) with a valid Senior Operator license shall be designated to assume the control room command function." With the SS assuming the dual role of SRO/Shift Technical Advisor, the above quoted parenthetical phrase would prohibit this individual (i.e., the only remaining SRO on duty) from assuming the control room command function. Therefore, the licensee has proposed to delete the parenthetical phrase "(other than the Shift Technical Advisor)." Since this change clarifies an inconsistency within the Technical Specifications and does not preclude an SRO from assuming the control room command function, the staff finds this acceptable.

- On March 25, 1987, 10 CFR Part 55 was amended to upgrade requirements for licensed operator selection, training and requalification. Subsequently, the NRC issued Revision 2 to Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants," which provides a method acceptable to the NRC staff for complying with 10 CFR Part 55. The requirements of 10 CFR Part 55 are more restrictive than Sections A and C of the enclosure to the March 18, 1980 NRC letter to all licensees currently listed in Sections 6.3 and 6.4 of the Technical Specifications. The licensee has proposed replacing the references to the March 18, 1980 letter with Revision 2 to Regulatory Guide 1.8. The proposed requirements are more restrictive than the current Technical Specification requirements and they address the requirements of 10 CFR Part 55. Therefore, the staff finds the proposed changes to Sections 6.3 and 6.4 acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Douglas V. Pickett, PDIV-2/NRR

Date: December 24, 1991