

April 28, 1994

Docket No. 50-482

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Mr. Neil S. Carns
 President and Chief Executive Officer
 Wolf Creek Nuclear Operating Corporation
 Post Office Box 411
 Burlington, Kansas 66839

Dear Mr. Carns:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 73 TO FACILITY
 OPERATING LICENSE NO. NPF-42 (TAC NO. M88053)

The Commission has issued the enclosed Amendment No. 73 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated October 21, 1993, as supplemented by letters dated March 14, 1994, and April 18, 1994.

The amendment revises Technical Specifications 6.5.1, Plant Safety Review Committee (PSRC) and 6.8, Procedures and Programs, in order to allow implementation of a Qualified Reviewer Program for the review and approval of new procedures and procedure changes.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

William D. Reckley, Project Manager
 Project Directorate IV-2
 Division of Reactor Projects III/IV
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 73 to NPF-42
2. Safety Evaluation

cc w/enclosures:
 See next page

* - See previous concurrence

Office	PDIV-2/LA	PDIV-2/PM	NRR:RPEB	NRR:OTSB	OGC	PDIV-2/P
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Date	4/19/94	4/19/94	3/24/94	3/25/94	4/12/94	4/28/94
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Mr. Neil S. Carns

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April 28, 1994

cc w/enclosures:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated October 21, 1993, and supplemented by letters dated March 14, 1994, and April 18, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 73, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

William D. Reekley for

Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

6-8
6-9
6-14
6-15

INSERT

6-8
6-9
6-14
6-15

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

- b. The position of Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
- c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.
- d. The position of Manager Operations shall hold or have previously held a senior reactor operator license for a similar unit (PWR).

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

- a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.
- b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Vice President Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Vice President Plant Operations shall designate in writing the Chairman and Alternate Chairman of the PSRC. PSRC membership shall include between six and eight additional members appointed by the Chairman and an additional member appointed by the Vice President Engineering. Selected members shall include, at a minimum, management responsible for the following areas of expertise: operations, maintenance, instrumentation and controls, chemistry, health physics and engineering. A single individual may cover multiple disciplines.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.**

**Except for the alternate for Engineering who is appointed by the Vice President Engineering.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) Administrative Control Procedures and changes thereto, and (2) procedures and changes thereto required by Specification 6.8.1 and requiring a 10 CFR 50.59 safety evaluation.
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Plant Operations, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Not used.
- j. Not used.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL and the Radwaste Treatment Systems, and
- l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President Plant Operations and to the Nuclear Safety Review Committee.
- m. Review of the Fire Protection Program and shall submit recommended changes to the Nuclear Safety Review Committee.

6.5.1.7 The PSRC shall:

- a. Recommend in writing to the Vice President Plant Operations approval or disapproval of items considered under Specification 6.5.1.6a. through d. and m. above,
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6b. through e. above constitutes an unreviewed safety question, and
- c. Provide written notification within 24 hours to the President and Chief Executive Officer and the Nuclear Safety Review Committee of disagreement between the PSRC and the Vice President Plant Operations; however, the Vice President Plant Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The PSRC shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President Plant Operations and the Nuclear Safety Review Committee.

6.5.2 NUCLEAR SAFETY REVIEW COMMITTEE (NSRC)

FUNCTION

6.5.2.1 The NSRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The NSRC shall report to and advise the President and Chief Executive Officer on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

COMPOSITION

6.5.2.2 The NSRC shall be composed of at least eight designated members, including the Chairman. Members of the NSRC may be from within the WCNOO organization or from outside organizations. The NSRC shall have sufficient expertise to adequately provide an independent review and audit of designated activities in the areas listed in Technical Specification Section 6.5.2.1. Additional members may be appointed by the Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRC Chairman to provide expert advice to the NSRC.

MEETING FREQUENCY

6.5.2.5 The NSRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6 The quorum of the NSRC necessary for the performance of the NSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least two-thirds of the NSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the Unit.

ADMINISTRATIVE CONTROLS

RECORDS (Continued)

- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the President and Chief Executive Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 of 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and submitted to the NSRC and the President and Chief Executive Officer.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The President and Chief Executive Officer and the NSRC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective ACTION taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the President and Chief Executive Officer within 14 days of the violation; and
- d. Critical operation of the Unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A, of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Section 7.1 of Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- e. Process Control Program implementation;
- f. ODCM implementation; and
- g. Quality Assurance Program implementation for effluent and environmental monitoring.
- h. Fire Protection Program implementation.

6.8.2 Each procedure of Specification 6.8.1 and changes thereto, and any other procedure or procedure change that the Vice President Plant Operations determines to affect nuclear safety, shall be reviewed and approved as described below, prior to implementation.

- a. Each procedure, or change thereto shall be reviewed by a Qualified Reviewer who is knowledgeable in the functional area affected, but is not the individual who prepared the procedure or procedure change. All required cross-disciplinary reviews of new procedures, procedure revisions or changes thereto shall be completed prior to approval.
- b. Procedures other than Administrative Control Procedures shall be approved by the responsible Manager or his designee as specified in Administrative Control Procedures. The Vice President Plant Operations shall approve Administrative Control Procedures. The Manager responsible for the Security Plan shall approve the Security Plan and implementing procedures. The Manager responsible for Emergency Planning shall approve the Radiological Emergency Response Plan and implementing procedures.
- c. The responsible Manager or his designee shall ensure each review includes a determination of whether a procedure, or change thereto, requires a 10 CFR 50.59 safety evaluation. If a procedure, or change thereto, requires a 10 CFR 50.59 safety evaluation, the responsible Manager or his designee shall forward the procedure, or change thereto with the associated 10 CFR 50.59 safety evaluation to the PSRC for review in accordance with Specification 6.5.1.6.a. Pursuant to Section 50.59, 10 CFR, NRC approval of items involving unreviewed safety questions shall be obtained prior to approval for implementation.
- d. Qualified Reviewers shall meet the applicable qualifications of ANSI/ANS 3.1-1978. Personnel recommended to be Qualified Reviewers shall be reviewed by the PSRC and approved and documented by the PSRC Chairman. The responsible Manager shall ensure that a sufficient complement of Qualified Reviewers for their functional area is maintained in accordance with Administrative Control Procedures.
- e. Records documenting the activities performed under Specification 6.8.2.a. through 6.8.2.d, shall be maintained in accordance with Specification 6.10.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.3 Temporary changes to procedures specified in 6.8.1 above may be made and implemented prior to obtaining the review and approval required in 6.8.2 above provided:

- a. The intent of the original procedure is not altered;
- b. Temporary changes shall, as a minimum, be approved by two cognizant members of the WCNO staff knowledgeable in the areas affected by the procedure. At least one of these shall be a member of WCNO supervision. Changes to operations procedures shall be approved by two cognizant members of WCNO staff knowledgeable in the areas affected by the procedure. One will hold a senior reactor operator license on the unit.
- c. The change is documented, reviewed and approved as required by 6.8.2, within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the appropriate portions of the Containment Spray System, Safety Injection System, Chemical and Volume Control System, RHR System, and the Nuclear Sampling System (PASS only). The program shall include the following:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,
- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective action for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. NPF-42
WOLF CREEK NUCLEAR OPERATING CORPORATION
WOLF CREEK GENERATING STATION
DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated October 21, 1993, and supplemented by letters dated March 14, 1994, and April 18, 1994, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specification Sections 6.5.1, Plant Safety Review Committee (PSRC) and 6.8, Procedures and Programs, in order to allow implementation of a Qualified Reviewer Program for the review and approval of new procedures and procedure changes.

2.0 EVALUATION

The proposed changes relate to the review responsibilities of the PSRC described in Technical Specification 6.5.1.6 and the processes associated with the review and approval of procedures described in Technical Specification 6.8. The current review responsibility of the PSRC includes all procedures and programs required by Technical Specification 6.8 and changes thereto. The proposed change would reduce the number of PSRC procedure reviews to only those involving administrative control procedures and those requiring a 10 CFR 50.59 safety evaluation. Changes to Technical Specification 6.8 replace the current procedure review and approval process with the requirements needed for implementation of a Qualified Reviewer Program. In addition, the proposed changes would delete items 6.5.1.6(i) and (j) related to PSRC reviews of the security plan and emergency plan.

The proposed Technical Specification 6.8.2 requires each new procedure or procedure change be reviewed by a Qualified Reviewer. A Qualified Reviewer is required to meet the applicable qualifications of ANSI/ANS 3.1-1978, "Standard for Selection and Training of Personnel for Nuclear Power Plants." The licensee's letter of March 14, 1994, provided a more precise reference to Section 4 of ANSI/ANS 3.1-1978 for the definition of the applicable qualifications for the Qualified Reviewers. Certification as a Qualified Reviewer follows PSRC review of an individual's qualifications and is approved by the PSRC Chairman.

Following review by a Qualified Reviewer, procedures, other than administrative control procedures, are approved by a responsible manager specified in the administrative control procedures. All required cross-disciplinary reviews of procedures or procedure changes are required to be completed prior to approval. Administrative control procedures and procedures requiring a 10 CFR 50.59 safety evaluation shall continue to receive PSRC review.

By letter dated March 14, 1994, the licensee provided additional information regarding the approval of procedures or procedure changes by the responsible manager or his designee. The licensee stated that the proposed technical specifications would allow a manager to delegate the responsibility for approving procedures to a member of his supervisory staff directly responsible for the procedure. The manager or designee approving a procedure or procedure change could not also be the initiator or reviewer of the procedure or procedure change. The letter addressed the staff's concern that the use of the term "or his designee" could potentially result in a lack of management involvement in the procedural process. The March 14, 1994, letter provided additional information and did not change the initial no significant hazards consideration determination published in the Federal Register on November 24, 1993.

The April 18, 1994, letter requested a change in the implementation period from 90 days to 120 days and did not change the initial no significant hazards consideration determination published in the Federal Register on November 24, 1993.

Temporary changes to procedures are governed by the proposed Technical Specification 6.8.3. Limitations on the use of temporary changes include: (1) the intent of the original procedure is not altered; (2) approval by at least two cognizant members of the licensee's staff, including at least one member of supervision, knowledgeable in the affected areas; (3) documentation, review and approval of the change as required by the proposed Qualified Reviewer process within 14 days of implementation. Temporary changes to operations procedures will require at least one of the two approving personnel to hold a senior reactor operator license. The above process differs from the recommendations of ANSI/ANS 3.2-1976, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants," Section 5.2.2, in that the standard does not exclude some temporary changes from requiring approval by a licensed senior reactor operator. However, the proposed change basically maintains the requirements of the existing Technical Specification 6.8.3 related to the review and approval of temporary procedure changes.

Section 13.5.1, Administrative Procedures, of the Standard Review Plan (SRP) states that the staff should determine that:

"the applicant has described the program and procedures that provide administrative controls over activities important to safety. These include the activities of the preparation, review, and approval of plant operating and maintenance procedures,..."

The existing system at Wolf Creek which requires PSRC review of procedures and procedure changes was found to be acceptable during the initial licensing reviews documented in NUREG-0881, "Safety Evaluation Report Related to the Operation of Wolf Creek Generating Station, Unit No. 1". The implementation of a Qualified Reviewer Program is intended to allow improved utilization of managers serving on the PSRC and other members of the unit staff and expedite the procedure review process. Details of the proposed Qualified Reviewer Program, including controlling administrative procedures and designation of personnel as qualified reviewers, will be reviewed and approved by the PSRC. The PSRC will also continue to review other administrative control procedures and all procedures and procedure changes requiring an evaluation in accordance with 10 CFR 50.59.

Several nuclear facilities have implemented qualified reviewer programs similar to that proposed by the licensee. The staff has determined that such programs may be adequate to provide administrative controls over activities important to safety related to the preparation, review, and approval of plant operating and maintenance procedures as required by applicable regulations and industry standards. The licensee's proposed program has been determined to satisfy the requirements of ANSI/ANS 3.2-1976 as endorsed by Regulatory Guide 1.33, Revision 2, "Quality Assurance Program Requirements (Operation)." The proposed Technical Specification 6.8.3 related to temporary changes was determined to remain essentially the same as the original technical specification requirements approved in NUREG-0881. The proposed changes to the technical specifications to allow implementation of a Qualified Reviewer Program are therefore acceptable.

In addition to the changes related to the implementation of the Qualified Reviewer Program, the licensee has proposed to delete the requirements for PSRC review of the security plan and emergency plan. The NRC issued guidance to all holders of operating licenses for nuclear power reactors on the proposed change by Generic Letter (GL) 93-07, "Modification of Technical Specification Administrative Control Requirements for Emergency and Security Plans" dated December 28, 1993. The licensee proposed modifying Technical Specification 6.5.1.6 by removing items (i) and (j), which reference the security and emergency plans, respectively, and their implementing procedures under the review responsibilities of the PSRC. The licensee has proposed technical specification changes that are consistent with the NRC guidance in GL 93-07 and that eliminate technical specification provisions that duplicate the requirements in 10 CFR Parts 50 and 73. The staff has reviewed this matter and finds that the proposed technical specification changes for Wolf Creek are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. D. Reckley

Date: April 28, 1994