

January 3, 1994

Docket No. 50-482

Mr. Neil S. Carns
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Carns:

DISTRIBUTION:

| | |
|----------------|------------|
| Docket File | JRoe |
| NRC PDR | GHill (2) |
| Local PDR | OPA |
| PDIV-2 Reading | CGrimes |
| EPeyton | ACRS (10) |
| WReckley (2) | OC/LFMB |
| EAdensam | DHagan |
| OGC | REckenrode |
| LYandell, RIV | GZech |

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 70 TO FACILITY
OPERATING LICENSE NO. NPF-42 (TAC NO. M88054)

The Commission has issued the enclosed Amendment No. 70 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated October 21, 1993.

The amendment revises Technical Specification 6.3.1 related to the qualification requirements for the position of Manager Operations and revises Technical Specification 6.5.1.2 to delete specific title designations from the Plant Safety Review Committee (PSRC) membership.

A clarification has been added to your proposed change to Technical Specification 6.5.1.2. The change consists of including a maximum number of ten PSRC members, including the Chairman, so that the quorum defined in Technical Specification 6.5.1.5, the Chairman and four members, continues to represent at least half of the committee. This change was discussed with your staff and was found to be acceptable.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original Signed By Suzanne C. Black for
William D. Reckley, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 70 to NPF-42
2. Safety Evaluation

*For previous concurrences
see attached ORC

cc w/enclosures:
See next page

Document Name: B:\M88054.AMD

| Office | PDIV-2/LA | PDIV-2/PM | NRR:RPEB | NRR:HHFB* | OGC | PDIV-2/D |
|--------|-----------|-------------|----------|------------|----------|----------|
| Name | EPeyton | WReckley:ye | GZech | REckenrode | | SBlack |
| Date | 12/23/93 | 12/23/93 | 12/27/93 | 12/08/93 | 12/27/93 | 12/29/93 |
| Copy | (Yes/No) | (Yes/No) | Yes/No | Yes/No | Yes/No | Yes/No |

9401130188 940103
PDR ADOCK 05000482
P PDR

NRC FILE CENTER COPY

1/10/94

Mr. Neil S. Carns

- 2 -

cc w/enclosures:

Jay Silberg, Esq.
Shaw, Pittman, Potts & Trowbridge
2300 N Street, NW
Washington, D.C. 20037

Mr. C. John Renken
Policy and Federal Department
Missouri Public Service Commission
P. O. Box 360
Jefferson City, Missouri 65102

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P. O. Box 311
Burlington, Kansas 66839

Chief Engineer
Utilities Division
Kansas Corporation Commission
1500 SW Arrowhead Road
Topeka, Kansas 66604-4027

Office of the Governor
State of Kansas
Topeka, Kansas 66612

Attorney General
1st Floor - The Statehouse
Topeka, Kansas 66612

County Clerk
Coffey County Courthouse
Burlington, Kansas 66839

Mr. Gerald Allen
Public Health Physicist
Bureau of Environmental Health Services
Division of Health
Kansas Department of Health
and Environment
109 SW Ninth
Topeka, Kansas 66612

Mr. Otto Maynard
Vice President, Plant Operations
Wolf Creek Nuclear Operating Corporation
P. O. Box 411
Burlington, Kansas 66839

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Mr. Keven J. Moles
Manager Regulatory Services
Wolf Creek Nuclear Operating Corporation
P. O. Box 411
Burlington, Kansas 66839



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated October 21, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9401130194 940103
PDR ADOCK 05000482
P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and is to be implemented within 30 days of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 3, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by amendment number and contains marginal lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE

6-7

INSERT

6-7

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

- b. The position of Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
- c. The NSRC members shall meet or exceed the requirements of ANSI/ANS 3.1-1981.
- d. The position of Manager Operations shall hold or have previously held a senior reactor operator license for a similar unit (PWR).

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Manager Training and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 with the following exceptions:

- a. The training program for Licensed Operators and Senior Operators shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1981 as endorsed by Regulatory Guide 1.8, Revision 2, and 10 CFR Part 55.
- b. Training shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Vice President Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Vice President Plant Operations shall designate in writing the Chairman and Alternate Chairman of the PSRC. PSRC membership shall include between six and eight additional members appointed by the Chairman and an additional member appointed by the Vice President Engineering. Selected members shall include, at a minimum, management responsible for the following areas of expertise: operations, maintenance, instrumentation and controls, chemistry, health physics and engineering. A single individual may cover multiple disciplines.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.**

**Except for the alternate for Engineering who is appointed by the Vice President Engineering.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Vice President Plant Operations to affect nuclear safety;
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Plant Operations, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Review of the plant Security Plan and implementing procedures and shall submit recommended changes to the NSRC;
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the NSRC;



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. NPF-42
WOLF CREEK NUCLEAR OPERATING CORPORATION
WOLF CREEK GENERATING STATION
DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated October 21, 1993, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Technical Specification 6.3.1 related to the qualification requirements for the position of Manager Operations and Technical Specification 6.5.1.2 related to the required composition of the Plant Safety Review Committee (PSRC). Technical Specification 6.3.1 has required members of the unit staff to meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978, "American National Standard for Selection and Training of Nuclear Power Plant Personnel." The proposed change would revise the Manager Operations qualification requirements from those listed in the standard (holding a senior reactor operator license) to requiring that the Manager Operations shall hold or have previously held a senior operator license for a similar unit. The revision to Technical Specification 6.5.1.2 involves replacement of the description of PSRC membership from a list of specific position titles to management responsible for various areas of expertise.

2.0 EVALUATION

Technical Specification 6.3.1 currently requires members of the licensee's staff to meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978, "American National Standard for Selection and Training of Nuclear Power Plant Personnel." ANSI/ANS 3.1-1978 Section 4.2.2 requires that at the time of initial core loading or appointment to the position, whichever is later, the operations manager shall hold a senior reactor operator license. The 1987 revision of ANSI/ANS 3.1 introduced additional flexibility by allowing the operations manager to hold a senior operator license, have held a senior operator license for a similar unit, or be certified as qualified by use of other criteria. ANSI/ANS 3.1-1987 also states that if the operations manager does not hold an NRC license, then the operations middle manager shall hold a senior reactor operator license.

9401130195 940103
PDR ADOCK 05000482
P PDR

Section 50.54(1) of Title 10 of the Code of Federal Regulations states those individuals responsible for directing the licensed activities of licensed operators shall be licensed as senior operators. In the case of Wolf Creek's current organization, this requires that the management level above the shift supervisors, the Supervisor Operations, hold a senior operator license. The licensee stated that in times of transition, when the Supervisor Operations position might be vacated, qualified personnel with a senior operator license would be available to fill the position. Technical Specification 6.2.2(g), which states that either the Supervisor Operations or Manager Operations shall hold a senior reactor operator license, would also ensure that the control room staff would continue to be supervised by a person with a senior operator license.

The proposed revision to Technical Specification 6.3.1 consists of adding an item (d) to the list of exceptions to the unit staff meeting the requirements of ANSI/ANS 3.1-1978. The proposed item (d) requires that the Manager Operations hold or have previously held a senior operator license for a similar unit (pressurized water reactor [PWR]). The licensee has stated that the Updated Safety Analysis Report (USAR) will be revised to reflect this change in commitment related to ANSI/ANS 3.1-1978.

The staff finds that the proposed Technical Specification 6.3.1 meets the relevant criteria in Standard Review Plan (SRP) Section 13.1.2-13.1.3, "Operating Organization" and 10 CFR 50.54 and is, therefore, acceptable.

The proposed change to Technical Specification 6.5.1.2, PSRC Composition, involves the replacement of a list of specific position titles with a paragraph describing the members as managers responsible for various areas of expertise. The required membership for the PSRC is reduced from the ten specific positions listed in the current Technical Specification to a proposed minimum of eight members comprised of the Chairman, designated by the Vice President Plant Operations, six members appointed by the chairman, and a member to be designated by the Vice President Engineering. The selected members are required to include management responsible for operations, maintenance, instrumentation and controls, chemistry, health physics, and engineering.

The proposed change to delete specific title designations from the technical specifications provides flexibility in appointment of members and also eliminates the need for amendment requests as a result of title changes. The replacement of the specific titles with functional areas ensures that the PSRC will provide interdisciplinary review of its subject matters. Qualification of personnel is ensured by the technical specification requirements for unit staff, including the managers serving on the PSRC, to meet or exceed the qualification requirements of ANSI/ANS 3.1-1978. Other technical specifications related to the PSRC, such as meeting frequency, quorum requirements, review responsibilities, and record requirements are not affected by the proposed change.

The staff added a clarification to the licensee's proposed change to Technical Specification 6.5.1.2. The change consists of including a maximum number of ten PSRC members, including the Chairman, so that the quorum defined in Technical Specification 6.5.1.5, the Chairman and four members, continues to represent at least half of the committee. The staff discussed this

clarification with the licensee and the licensee agreed to the staff's requested change. The change does not change the initial no significant hazards consideration determination published in the Federal Register on November 24, 1993.

The staff has reviewed the proposed change, as amended, and finds that the criteria of SRP Section 13.4 continue to be met. The change is, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. D. Reckley

Date: January 3, 1994