PROPRIETARY INFORMATION

June 28, 2002

Mr. James F. Klapproth, Manager
Engineering & Technology
GE Nuclear Energy
175 Curtner Ave
San Jose, CA 95125

SUBJECT: REVIEW OF GE NUCLEAR ENERGY LICENSING TOPICAL REPORT, NEDC-32721P, "APPLICATION METHODOLOGY FOR GE STACKED DISC SUCTION STRAINER," PART II (TAC NO. MB3311)

Dear Mr. Klapproth:

By letters dated April 3, 1997, November 21, 1997, and October 30, 2001, GE Nuclear Energy (GENE) submitted Topical Report NEDC-32721P, "Application Methodology for GE Stacked Disk Suction Strainer," to the NRC for review. On February 3, 1999, the staff issued Part I of its safety evaluation of GENE's methodology for determining strainer performance. The NRC staff has completed its review of the GENE methodology for determining the hydrodynamic load inputs to the structural analyses of the new emergency core cooling system (ECCS) strainers. Based on the review, we have concluded that the proposed scaling factors are acceptable for use in the determination of the hydrodynamic load inputs to the structural analyses. Therefore, the NRC staff concluded that licensees with GE stacked disk strainers will not exceed the strainer and containment penetration design loads following a safety/relief valve (SRV) discharge or loss-of-coolant accident event using the proposed methodology.

The staff finds that the subject topical report is acceptable for referencing in licensing applications to the extent specified under the limitations delineated in the report and in the associated NRC safety evaluation. The enclosed safety evaluation defines the basis for acceptance of the topical report.

The NRC requests that GENE publish an accepted version of the revised Topical Report NEDC-32721P within 3 months of receipt of this letter. The accepted version shall incorporate this letter and the enclosed safety evaluation between the title page and the abstract, and add a "-A" (designating accepted) following the report identification number (i.e., NEDC-32721P-A).

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If the NRC's criteria or regulations change so that its conclusion in this letter that the topical report is acceptable is invalidated, GENE and/or the applicant referencing the topical report will be expected to revise and resubmit its respective documentation, or submit justification for the continued applicability of the topical report without revision of the respective documentation.

Pursuant to 10 CFR 2.790, we have determined that the safety evaluation provided as Enclosure 1 contains proprietary information. We have prepared a non-proprietary version of the safety evaluation (Enclosure 2) that we have determined does not contain proprietary information. However, we will delay placing Enclosure 2 in the public document room for a period of ten (10) working days from the date of this letter to provide you with the opportunity to comment on the proprietary aspects only. If you believe that any information in Enclosure 2 is proprietary, please identify such information line by line and define the basis pursuant to the criteria of 10 CFR 2.790.

If you have any questions, please contact Alan Wang, GENE Project Manager, at (301) 415-1445.

Sincerely,

/RA/

Cornelius F. Holden, Jr., Acting Director
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Project No. 710

Enclosures: 1. Proprietary Safety Evaluation
2. Non-Proprietary Safety Evaluation

cc w/encl. 2: See next page

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Project Directorate IV
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2. Non-Proprietary Safety Evaluation

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