

July 18, 1991

Docket No. 50-482

DISTRIBUTION:

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Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 45 TO FACILITY
OPERATING LICENSE NO. NPF-42 (TAC NO. 79921)

The Commission has issued the enclosed Amendment No. 45 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated March 5, 1991 (ET 91-0045).

The amendment revises Section 6.0 of the Wolf Creek Technical Specifications to reflect an organizational change and various title changes in the Wolf Creek Nuclear Operating Corporation.

It should be noted that copies of Amendment No. 45, transmitted by letter dated June 27, 1991, contained administrative errors on Technical Specification page 6-9 and were distributed to those individuals listed on page 2 of this letter. This copy of Amendment No. 45 has been corrected and copies are being sent to those individuals listed on the next page. We apologize for any confusion or inconvenience this may have caused.

A copy of our related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

Douglas V. Pickett, Project Manager
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

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P PDR

Enclosures:

- 1. Amendment No. 45 to NPF-42
- 2. Safety Evaluation

cc w/enclosures:

See next page

*For previous concurrences see attached ORC

NRC FILE CENTER COPY

OFC	: PDIV-2-2/LA	: PDIV-2/PM	: OGC*	: PDIV-2/(A)D	:
NAME	: EPeyton	: DPickett	: CWoodhead	: GDick	:
DATE	: 7/18/91	: 7/18/91	: 5/28/91	: 7/18/91	:

OFFICIAL RECORD COPY
Document Name: WC CORRECTION LETTER

Handwritten signature and initials

July 18, 1991

cc w/enclosures:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated March 5, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

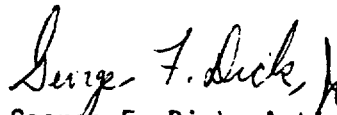
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 45 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick, Acting Director
Project Directorate IV-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 27, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

6-1
6-7
6-8
6-9
6-14
6-15
6-23
6-24

INSERT

6-1
6-7
6-8
6-9
6-14
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6-23
6-24

ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Director Plant Operations shall be responsible for overall Unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Supervising Operator, under the Shift Supervisor, shall be responsible for the control room command function. A management directive to this effect, signed by the President and Chief Executive Officer shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 Onsite and Operating Corporation Organization

Onsite and operating corporation organizations shall be established for unit operation and corporate management, respectively. The onsite and operating corporation organizations shall include the positions for the activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or equivalent forms of documentation. These requirements shall be documented in the Updated Safety Analysis Report.
- b. The Director Plant Operations shall be responsible for overall unit safe operation and shall have control of those onsite activities necessary for safe operation and maintenance of the plant.
- c. The President and Chief Executive Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out the health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 Unit Staff

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;

ADMINISTRATIVE CONTROLS

Unit Staff (Continued)

- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the Unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the control room;
- c. An individual from the Health Physics Group*, qualified in radiation protection procedures, shall be on site when fuel is in the reactor;
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least 5 members* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Supervisor, and the two other members of the minimum shift crew necessary for safe shutdown of the Unit and any personnel required for other essential functions during a fire emergency; and
- f. Administrative procedures shall be developed and implemented to limit the working hours of Unit Staff who perform safety-related functions; e.g., Senior Operators, Operators, Health Physicists, Auxiliary operators, and key maintenance personnel.

The amount of overtime worked by Unit Staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

- g. The Operations Supervisor shall hold a senior reactor operator license.

*May be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5 of ANSI/ANS 3.1-1978 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the ISEG or another plant group.

6.5 REVIEW AND AUDIT

6.5.1 PLANT SAFETY REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.1.1 The PSRC shall function to advise the Director Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PSRC shall be composed of the:

Member:	Manager Nuclear Plant Engineering Wolf Creek
Member:	Manager Operations
Member:	Manager Technical Support
Member:	Manager Maintenance and Modifications
Member:	Manager Instrumentation and Control
Member:	Supervisor Reactor Engineering
Member:	Manager Radiation Protection
Member:	Manager Chemistry
Member:	Supervisor Results Engineering
Chairman:	Manager Plant Support

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PSRC shall be responsible for:

- a. Review of: (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Director Plant Operations to affect nuclear safety;
- b. Review of all proposed changes, tests and experiments which may involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Review of all proposed changes to Technical Specifications or the Operating License;
- d. Review of all safety evaluations performed under the provision of Section 50.59(a)(1), 10 CFR, for changes, tests and experiments;
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President Operations, and to the Nuclear Safety Review Committee (NSRC);
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of operating abnormalities, deviations from expected performance of plant equipment and of unanticipated deficiencies in the design or operation of structures, systems or components that affect nuclear safety;
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman, NSRC;
- i. Review of the plant Security Plan and implementing procedures and shall submit recommended changes to the NSRC;
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the NSRC;

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL and the Radwaste Treatment Systems, and
 - l. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Director Plant Operations and to the Nuclear Safety Review Committee.
 - m. Review of the Fire Protection Program and shall submit recommended changes to the Nuclear Safety Review Committee.
- 6.5.1.7 The PSRC shall:
- a. Recommend in writing to the Director Plant Operations approval or disapproval of items considered under Specification 6.5.1.6a. through d. and m. above,
 - b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.1.6a. through e. above constitutes an unreviewed safety question, and
 - c. Provide written notification within 24 hours to the Vice President Operations and the Nuclear Safety Review Committee of disagreement between the PSRC and the Director Plant Operations; however, the Director Plant Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The PSRC shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility provisions of these Technical Specifications. Copies shall be provided to the Vice President Operations and the Nuclear Safety Review Committee.

6.5.2 NUCLEAR SAFETY REVIEW COMMITTEE (NSRC)

FUNCTION

- 6.5.2.1 The NSRC shall function to provide independent review and audit of designated activities in the areas of:
- a. Nuclear power plant operations,
 - b. Nuclear engineering,
 - c. Chemistry and radiochemistry,
 - d. Metallurgy,
 - e. Instrumentation and control,
 - f. Radiological safety,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

The NSRC shall report to and advise the President and Chief Executive Officer on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

COMPOSITION

6.5.2.2 The NSRC shall be composed of at least eight designated members, including the Chairman. Members of the NSRC may be from within the WCNOG organization or from outside organizations. The NSRC shall have sufficient expertise to adequately provide an independent review and audit of designated activities in the areas listed in Technical Specification Section 6.5.2.1. Additional members may be appointed by the Chairman.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NSRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NSRC Chairman to provide expert advice to the NSRC.

MEETING FREQUENCY

6.5.2.5 The NSRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6 The quorum of the NSRC necessary for the performance of the NSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least two-thirds of the NSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the Unit.

ADMINISTRATIVE CONTROLS

RECORDS (Continued)

- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the President and Chief Executive Officer and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 of 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and submitted to the NSRC and the President and Chief Executive Officer.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The President and Chief Executive Officer and the NSRC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective ACTION taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRC and the President and Chief Executive Officer within 14 days of the violation; and
- d. Critical operation of the Unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A, of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Section 7.1 of Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- e. Process Control Program implementation;
- f. ODCM implementation; and
- g. Quality Assurance Program implementation for effluent and environmental monitoring.
- h. Fire Protection Program implementation.

Major Procedures, supported by appropriate Minor Procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, etc.), shall be provided for the above activities.* A Major Procedure is a procedure which controls safety-related activities, and establishes one or more basic controls, overall responsibilities, authority assignments or administrative and operational ground rules at the Wolf Creek plant. Major Procedures are written to meet the requirements of ANSI N18.7-1976/ANS 3.2 and generally are supported by Minor Procedures which provide delineation of details such as for valve lineups, calibration procedures, operating instructions, data sheets, alarm responses, and other procedures identified as "supporting." Major Procedures require signature approval in all cases by the Director Plant Operations or a Call Superintendent in his absence. A Minor Procedure is a procedure which controls safety-related activities in support of a Major Procedure. It addresses a specific topic or sub-topic established by its 'parent' Major Procedure, expanding on it by providing working level instructions. Minor Procedures are not permitted to contradict requirements contained in their governing Major Procedure. Minor Procedures require signature approval by the Director Plant Operations, or a Call Superintendent in his absence, only at Revision '0.'

6.8.2 Approval of Procedures

- a. All Major Procedures of the categories listed in Specification 6.8.1 and modifications to the intent thereof shall be reviewed by the PSRC and approved by the Director Plant Operations prior to implementation and reviewed periodically as set forth in Administrative Procedures.
- b. Minor Procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry and analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be approved by the PSRC or a Subcommittee thereof.
- c. Corporate Emergency Plan implementing procedures shall be reviewed by appropriate corporate and plant personnel and approved by the President and Chief Executive Officer as set forth in General Procedures.

*With the exception of Corporate Emergency Plan implementing procedures. Corporate Emergency Plan implementing procedures shall be provided but shall not be designated as major or minor procedures.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.3 Changes to Procedures

- a. Temporary changes to Major Procedures, of the categories listed in Specification 6.8.1 which do not change the intent or generate an unreviewed safety question of the original or subsequent approved procedure, may be made provided such changes to operating procedures are approved by the Shift Supervisor (SRO licensed) and one of the Call Superintendents. For temporary changes to Major Procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, or Health Physics which do not change the intent or generate an unreviewed safety question, changes may be made upon approval of the Cognizant Group Leader and a Call Superintendent.

All temporary changes to Major Procedures (made by a Call Superintendent and either a Cognizant Group Leader or the Shift Supervisor) shall subsequently be reviewed by the PSRC and approved by the Director Plant Operations within 14 days, except that temporary changes to Major Procedures made during a refueling outage may be reviewed and approved at any time prior to initial criticality of the reload core. All permanent changes to Major Procedures shall be made in accordance with Specification 6.8.2.a.

- b. All temporary or permanent changes to Minor Operating Procedures (checkoff lists, alarm responses, data sheets, operating instructions, etc.) shall be approved by the Shift Supervisor, and shall be subsequently reviewed and approved by the Operations PSRC Subcommittee. All temporary or permanent changes to other Minor Procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, Chemistry, or Health Physics, shall be approved by a Cognizant Group Leader and shall be subsequently reviewed and approved by the appropriate PSRC Subcommittee.
- c. Temporary changes to Corporate Emergency Plan implementing procedures may be made provided that: (1) the intent of the original procedure is not altered, (2) the change is approved by the Manager Technical Services, and (3) the change is documented, reviewed by appropriate Corporate and plant personnel and approved by the President and Chief Executive Officer within 14 days of the implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the appropriate portions of the Containment Spray System, Safety Injection System, Chemical and Volume Control System, RHR System, and the Nuclear Sampling System (PASS only). The program shall include the following:

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,
- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective action for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.

ADMINISTRATIVE CONTROLS

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to Paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by Paragraph 20.203(c)(2), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area, or
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them, or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Manager Radiation Protection in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Supervisor/Supervising Operator on duty and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work areas and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as closed-circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities being performed within the area.

ADMINISTRATIVE CONTROLS

HIGH RADIATION AREA (Continued)

For individual high radiation areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and where no enclosure can be reasonably constructed around the individual area, that individual area shall be barricaded, conspicuously posted, and a flashing light shall be activated as a warning device.

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Director Plant Operations.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Director Plant Operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated March 5, 1991, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed changes would revise Section 6.0 of the technical specifications to reflect an organizational change and various title changes in the Wolf Creek Nuclear Operating Corporation.

2.0 EVALUATION

A recent organizational change in the Wolf Creek Nuclear Operating Corporation has resulted in a number of title changes along with some changes in responsibility. Some of the title changes have resulted in the need to revise Section 6.0 of the Wolf Creek Technical Specifications. The changes that will be reflected in the technical specifications are described as follows:

- a. The title of Vice President Nuclear Operations has been changed to Vice President Operations. This title change is administrative in nature and does not represent any change in reporting relationships, job responsibilities, or overall organizational commitments.
- b. The title of Plant Manager has been changed to Director Plant Operations. This title change is administrative in nature and does not represent any change in reporting relationships, job responsibilities, or overall organizational commitments.
- c. Technical Specification 6.5.1.2, which identifies the composition of the Plant Safety Review Committee, has been revised to reflect additional title changes. The title of Health Physicist has been changed to Manager Radiation Protection and the title of Chemist has been changed to Manager Chemistry. These title changes are administrative in nature and do not represent any change in reporting relationships, job responsibilities, or overall organizational commitments. Also, minor, miscellaneous changes have been made to Specification 6.5.1.2 to reflect actual titles that have previously existed. Examples of these latter changes include changing "Manager of Operations" to "Manager Operations" and "Reactor Engineering Supervisor" to "Supervisor Reactor Engineering."

- d. The title of Site Health Physicist in Specification 6.12.1 has been changed to Manager Radiation Protection. As discussed above, this title change is administrative in nature and does not represent any change in reporting relationships, job responsibilities, or overall organizational commitments.
- e. The title of Emergency Planning Coordinator in Specification 6.8.3.c has been changed to Manager Technical Services. This title change represents an organizational change that the licensee describes as increasing management effectiveness through the consolidation of several departments with similar responsibilities.

The changes described above do not represent any changes that would adversely affect or endanger the health or safety of the general public or involve a significant safety hazard. The changes are administrative in nature and do not change plant operations or any commitments previously made by the licensee. Therefore, the staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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