March 19, 1993

Docket No. 50-482

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Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. M85269)

The Commission has issued the enclosed Amendment No. 60 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated December 22, 1992.

The amendment deletes the surveillance requirement of Technical Specification 4.8.1.1.2(i)(2), which involves performing a pressure test of portions of the emergency diesel fuel oil system. Alternative testing for the tanks and piping would include leak testing at hydrostatic head pressure with the tanks filled to design capacity and would be governed by Technical Specification 4.0.5.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original Signed By

William D. Reckley, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 60 to NPF-42
2. Safety Evaluation

cc w/enclosures:
See next page

OGC 3-10-93

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Date	2/11 /93	2/11 /93	[redacted]	2/23/93	2/ /93	3/18/93

WDR for

#50 #23 JNH 3/4

Document Name: M85269.WC

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P PDR

NRC FILE COPY

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated December 22, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 60, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

William D Reckley for

Suzanne C. Black, Director
Project Directorate IV-2
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 19, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by amendment number and contains marginal lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE	INSERT
3/4 8-6a	3/4 8-6a

SURVEILLANCE REQUIREMENTS (Continued)

i. At least once per 10 years by:

1) Draining each fuel oil storage tank, removing the accumulated sediment and cleaning the tank using a sodium hypochlorite solution or equivalent.

4.8.1.1.3 Reports - All diesel generator failures, valid or nonvalid, shall be reported in a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days. Reports of diesel generator failures shall include the information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977. If the number of failures in the last 100 valid tests (on a per nuclear unit basis) is greater than or equal to 7, the report shall be supplemented to include the additional information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

1.0 INTRODUCTION

By application dated December 22, 1992, Wolf Creek Nuclear Operating Corporation (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. NPF-42) for the Wolf Creek Generating Station. The proposed amendment deletes the surveillance requirement of Technical Specification 4.8.1.1.2(i)(2), which involves performing a pressure test of portions of the emergency diesel fuel oil system. Alternative testing for the tanks and piping would include leak testing at hydrostatic head pressure with the tanks filled to design capacity and would be governed by Technical Specification 4.0.5.

2.0 EVALUATION

The existing Technical Specification 4.8.1.1.2(i)(2) requires a pressure test of those portions of the diesel fuel oil system designed to Section III, subsection ND of the ASME Boiler and Pressure Vessel Code. This pressure test is required to be performed at 110 percent of the system design pressure. This test is consistent with the acceptable method which is presented in NRC Regulatory Guide 1.137, "Fuel-Oil Systems for Standby Diesel Generators". Regulatory Guide 1.137 adopts Section 7.3 of ANSI N195-1976, "Fuel Oil Systems for Standby Diesel-Generators" which specifies testing in accordance with Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components", of the ASME Code. Section XI Article IWD-5000 in turn requires testing of components at 1.10 times the system pressure for systems with design temperatures less than 200°F or in the case of atmospheric storage tanks, the hydrostatic head, developed with the tank filled to its design capacity, is considered to be an acceptable test pressure.

The licensee has requested the proposed change due to the diesel fuel oil system's incompatibility with the requirements of the performance of a pressure test at 110 percent of system pressure. The diesel fuel oil tank is vented to atmosphere without an existing ability to isolate and pressurize the tank in order to perform the pressure test. The fulfillment of ASME Code requirements is achieved by the performance of the alternate testing consisting of leak testing with the associated atmospheric tanks filled to design capacity. The alternate test is considered to provide equivalent assurance of the tank and piping integrity in that filling the tank to design

capacity and verifying that no loss of inventory occurs is comparable to pressurizing the system and monitoring for any decrease in pressure. The diesel fuel oil system is classified as ASME Code Class 3 in accordance with the guidance of Regulatory Guide 1.26, "Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants." Technical Specification 4.0.5 requires testing of ASME Code Class 1, 2, and 3 components in accordance with Section XI of the ASME Code. The existing Technical Specification 4.8.1.1.2(i)(2) is therefore seen as redundant to the requirements of Technical Specification 4.0.5 in that ASME Section XI testing is required, but the specific testing methodology of Technical Specification 4.8.1.1.2(i)(2) is not practical for the current design of the diesel fuel oil system.

Upon review of the licensee's submittal, the staff concludes that the alternative testing allowed by Section XI of the ASME Code is an adequate inservice testing methodology and that the performance of the inservice testing is required by Technical Specification 4.0.5. Therefore, the deletion of Technical Specification 4.8.1.1.2(i)(2) is deemed acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Kansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 7009). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: William D. Reckley

Date: March 19, 1993