

November 1, 1989

Docket No. 50-482

Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

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Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO.34 TO FACILITY
OPERATING LICENSE NO. NPF-42 (TAC NO. 74108)

The Commission has issued the enclosed Amendment No. 34 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated July 25, 1989.

The amendment revises Technical Specification Surveillance Requirements 4.9.6.1 and 4.9.6.2 to change the times when the Refueling Machine and the Auxiliary Hoist must be demonstrated operable.

A copy of our related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/
Douglas V. Pickett, Project Manager
Project Directorate IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

- Amendment No. 34 to License No. NPF-42
- Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
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Douglas V. Pickett, Project Manager
Project Directorate IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 34 to
License No. NPF-42
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Bart D. Withers
Wolf Creek Nuclear Operating Corporation

Wolf Creek Generating Station
Unit No. 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated July 25, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 34, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 1, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE PAGE

3/4 9-7

INSERT PAGE

3/4 9-7

REFUELING OPERATIONS

SURVEILLANCE REQUIREMENTS (Continued)

to the movement of fuel assemblies in the reactor vessel by performing a load test of at least 125% of the secondary automatic overload cutoff and demonstrating an automatic load cutoff when the refueling machine load exceeds the Setpoints of Specification 3.9.6a.2) and by demonstrating an automatic load reduction trip when the load reduction exceeds the Setpoint of Specification 3.9.6a.3).

4.9.6.2 Each auxiliary hoist and associated load indicator used for movement of drive rods within the reactor vessel shall be demonstrated OPERABLE within 100 hours prior to the movement of drive rods within the reactor vessel by performing a load test of at least 1250 pounds.

REFUELING OPERATIONS

3/4.9.7 CRANE TRAVEL - SPENT FUEL STORAGE FACILITY

LIMITING CONDITION FOR OPERATION

3.9.7 Loads in excess of 2250 pounds shall be prohibited from travel over fuel assemblies in the spent fuel storage facility.

APPLICABILITY: With fuel assemblies in the spent fuel storage facility.

ACTION:

- a. With the requirements of the above specification not satisfied, place the crane load in a safe condition.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.7 Crane interlocks and physical stops which prevent crane travel with loads in excess of 2250 pounds over fuel assemblies shall be demonstrated OPERABLE within 7 days prior to crane use and at least once per 7 days thereafter during crane operation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

INTRODUCTION

Technical Specification Surveillance Requirements 4.9.6.1 and 4.9.6.2 currently require the Refueling Machine used for movement of fuel assemblies and the Auxiliary Hoist and associated load indicator used for movement of drive rods to be demonstrated operable within 100 hours prior to removal of the Reactor Vessel Head.

By letter dated July 25, 1989, the licensee proposed technical specification changes that would revise Surveillance Requirement 4.9.6.1 to require the Refueling Machine to be demonstrated operable within 100 hours prior to the movement of fuel assemblies within the Reactor Vessel. The proposed change would also revise Surveillance Requirement 4.9.6.2 to require the Auxiliary Hoist to be demonstrated operable within 100 hours prior to the movement of drive rods within the Reactor Vessel.

EVALUATION

The sequence of events for fuel movement during a refueling outage is as follows:

- 1) Strip vessel head of support equipment.
- 2) Detension vessel head studs and remove them to storage area.
- 3) Lift vessel head and place it in its storage location.
- 4) Raise reactor coolant level and refueling pool level up to 23 feet above the fuel assemblies and unlatch the control rods, using the Auxiliary Hoist.
- 5) Level is then raised to 23 feet above the reactor vessel flange and the upper internals are removed from the reactor vessel and placed in its storage location.
- 6) Fuel assemblies are then removed from the reactor vessel using the Refueling Machine (Manipulator Crane).

The above chronology shows that neither the Auxiliary Hoist nor the Refueling Machine are needed prior to lifting the reactor vessel head. When the reactor vessel head studs are being detensioned (step 2), the reactor coolant level is maintained at least one foot below the flange level. Operations during these conditions of reduced reactor coolant system inventory is of concern to the staff and has been addressed in Generic Letters 87-12, "Loss of Residual Heat Removal (RHR) While the Reactor Coolant System (RCS) is Partially Filled," and

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88-17, "Loss of Decay Heat Removal." Both of these generic letters discuss the risks of losing decay heat removal systems during periods of reduced reactor coolant system inventory.

In practice, the operability tests of the Auxiliary Hoist and the Refueling Machine are typically conducted during this time of reduced inventory. This has resulted in 1) delays in lifting the reactor vessel head while performing surveillance requirements, and 2) the likelihood of an extended duration in a reduced inventory situation if problems develop during the operability tests.

Once the reactor vessel head is lifted, the reactor coolant inventory is increased to the control rod unlatch level. This is a considerable increase in coolant inventory and it ameliorates the staff's concern of operating in a reduced inventory. If the operability tests for the Auxiliary Hoist and Refueling Machine were scheduled as proposed by the licensee, they could be performed after lifting the reactor vessel head thus minimizing plant operations in a reduced coolant inventory.

Considering that 1) there is no practical need for performing the subject operability tests prior to lifting the vessel head, and 2) scheduling the operability tests prior to system need minimizes plant operations during reduced coolant inventory and thus increases overall plant safety, the staff finds the licensee's proposal acceptable.

ENVIRONMENTAL CONSIDERATION

The amendment involves a change in a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 1, 1989

Principal Contributor: Douglas V. Pickett