March 28, 1990

Docket No. 50-482

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Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO.37 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. 75930)

The Commission has issued the enclosed Amendment No.37 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated February 7, 1990 (WM 90-0026).

The amendment allows operational mode changes while certain control room ventilation Technical Specification action statements are in effect. However, with regard to your request to institute a consistent allowed outage time of seven days prior to placing the system in its emergency operational mode, the Commission is deferring action pending development of the staff's Technical Specification Improvement Program.

A copy of our related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/
Douglas V. Pickett, Project Manager
Project Directorate IV
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 37 to License No. NPF-42
2. Safety Evaluation

cc w/enclosures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 28, 1990

Docket No. 50-482

Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

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Sincerely,

Douglas V Pickett

Douglas V. Pickett, Project Manager
Project Directorate IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 37 to
License No. NPF-42
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. NPF-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated February 7, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendix A, as revised through Amendment No. 37 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Frederick J. Hebdon, Director
Project Directorate IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 28, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 37

FACILITY OPERATING LICENSE NO. NPF-42

DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marg lines indicating the area of change. The corresponding overleaf pages a provided to maintain document completeness.

REMOVE PAGES	PAGES
3/4 3-21	3/4
3/4 7-14	3/4 7-14

TABLE 3.3-3 (Continued)

ACTION STATEMENTS (Continued)

b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 2 hours for surveillance testing of other channels per Specification 4.3.2.1.

ACTION 20 - With less than the Minimum Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.

ACTION 21 - With the number of OPERABLE Channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.

ACTION 22 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.

ACTION 23 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the action required by Specification 3.7.1.5.

ACTION 24 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, declare the affected auxiliary feedwater pump inoperable and take the ACTION required by Specification 3.7.1.2.

ACTION 25 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, declare the affected diesel generator and off-site power source inoperable and take the ACTION required by Specification 3.8.1.1.

ACTION 26 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or initiate and maintain operation of the Control Room Emergency Ventilation System. During operation in MODE 5 and 6, the provisions of Specification 3.0.4 are not applicable.

ACTION 27 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.

TABLE 3.3-4

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
1. Safety Injection (Reactor Trip, Phase "A" Isolation, Feedwater Isolation, Component Cooling Water, Turbine Trip, Auxiliary Feedwater-Motor-Driven Pump, Emergency Diesel Generator Operation, Containment Cooling, and Essential Service Water Operation)					
a. Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays (SSPS)	N.A.	N.A.	N.A.	N.A.	N.A.
c. Containment Pressure High-1	3.6	0.71	1.98	$\leq$ 3.5 psig	$\leq$ 4.5 psig
d. Pressurizer Pressure - Low	16.2	10.71	2.49	$\geq$ 1830 psig	$\geq$ 1815 psig
e. Steam Line Pressure - Low	19.6	14.81	1.93	$\geq$ 615 psig	$\geq$ 571 psig*

3/4.7.5 ULTIMATE HEAT SINK

LIMITING CONDITION FOR OPERATION

3.7.5 The ultimate heat sink (UHS) shall be OPERABLE with:

a. The crest of the UHS dam below the rip rap cover and corresponding water level at or above elevation 1070 Mean Sea Level, USGS datum, and

b. The plant inlet water temperature of less than or equal to 90°F.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.5 The UHS shall be determined OPERABLE:

a. At least once per 24 hours by verifying the above required water temperature and water level to be within their limits, and

b. At least once per 12 months by verifying that the crest of the UHS dam below the rip rap cover is at or above elevation 1070 Mean Sea Level, USGS datum.

3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.6 Two independent Control Room Emergency Ventilation Systems shall be OPERABLE.

APPLICABILITY: All MODES.

ACTION:

MODES 1, 2, 3 and 4:

With one Control Room Emergency Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5 and 6:

a. With one Control Room Emergency Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or initiate and maintain operation of the remaining OPERABLE Control Room Emergency Ventilation System in the recirculation mode. The provisions of Specification 3.0.4 are not applicable.

b. With both Control Room Emergency Ventilation Systems inoperable, or with the OPERABLE Control Room Emergency Ventilation System, required to be in the recirculation mode by ACTION a., not capable of being powered by an OPERABLE emergency power source, suspend all operations involving CORE ALTERATIONS or positive reactivity changes.

SURVEILLANCE REQUIREMENTS

4.7.6 Each Control Room Emergency Ventilation System shall be demonstrated OPERABLE:

a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 84°F;

b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers of both the Filtration and Pressurization Systems and verifying that the Pressurization System operates for at least 10 continuous hours with the heaters operating;

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-42

WOLF CREEK NUCLEAR OPERATING CORPORATION

WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

INTRODUCTION

The Control Room Emergency Ventilation System (CREVS) provides the control room with a conditioned atmosphere following various Design Basis Accidents (DBAs) such as Loss of Coolant Accident (LOCA), fuel handing accident, rod ejection, main steamline break and steam generator tube rupture. This system ensures that the instrumentation and equipment located in the control room will be maintained within their design temperatures and that the control room will remain habitable.

The Control Room Emergency Ventilation System consists of two separate and redundant trains which recirculate the control room air. The system initiates filtered ventilation of the control room following receipt of an actuation signal. The CREVS design basis is established by the consequences of the limiting DBA which is a LOCA in MODE 1, 2, 3 and 4 and a fuel handling accident in MODE 5 and 6. The LOCA analysis assumes that only one train of the CREVS is functional due to a single failure which disables the other train. The design basis is also established by the potential chlorine gas sources which would have the potential for incapacitating personnel in the control room if mitigating actions were not taken.

By letter dated February 7, 1990, the licensee identified inconsistencies in the Technical Specifications (TSs) covering the CREVS and its associated actuation instrumentation. These TSs are 3.3.2, "Engineered Safety Features Actuation System Instrumentation", 3.3.3.1, "Radiation Monitoring for Plant Operations", 3.3.3.7, "Chlorine Detection System", and 3.7.6, "Control Room Emergency Ventilation System."

The licensee's letter requested an amendment to revise the action statements of TSs 3.3.2 and 3.7.6 concerning the CREVS and its associated actuation instrumentation such that an exception to TS 3.0.4 is allowed for those action statements which permit continued unit operation for an unlimited period of time. The licensee stated that entry into one of the above action statements (i.e., when a limiting condition for operation (LCO) is not met and designated actions are required under TS 3.3.2 and 3.7.6) as currently written would restrict operating mode changes due to TS 3.0.4. This could result in a needless delay in a mode change since most of the action statements allow continued unit operation for an unlimited time. The licensee, therefore, proposes the change on the basis that the action statement establishes an acceptable level of safety for continual unit operation, hence, mode changes need not be restricted.

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In addition, the licensee has requested amending the allowed outage time for an inoperable CREVS train or associated actuation instrumentation before the remaining train must be placed in its emergency recirculation mode. The licensee has requested revising TSs 3.3.2 and 3.3.3.1 to establish a consistent allowed outage time of 7 days.

EVALUATION

The action statements for each of the four TSs identified above permit unlimited plant operation provided that the CREVS is placed in its emergency recirculation mode. Inconsistencies identified by the licensee include the exception to TS 3.0.4 regarding mode changes while in a TS action statement. Exceptions to TS 3.0.4 currently exist for TS 3.3.3.1 and 3.3.3.7 but do not exist for 3.3.2 and 3.7.6. In addition, the allowed outage time for a single CREVS train or associated actuation instrumentation is 7 days for TSs 3.3.3.7 and 3.7.6 but only 48 hours for TS 3.3.2 and 72 hours for TS 3.3.3.1.

The proposed change regarding TS 3.0.4 would allow the licensee to make operation mode changes in modes 5 and 6 while operating in accordance with the existing action statement, which allows continued operation for an unlimited time period. This change is consistent with the guidance provided in Generic Letter 87-09, which acknowledges that it is overly conservative to restrict operational mode changes under conditions which provide an acceptable level of safety for unlimited continued operation. This proposed change will thereby eliminate a situation that could result in a delay in plant startup. Based on the guidance provided in the generic letter, the staff finds these modifications as proposed by the licensee acceptable.

Establishing a uniform allowed outage time of 7 days for a CREVS train or for CREVS actuation instrumentation is desirable for consistency between the various TSs. However, the staff is currently in the process of modifying the Westinghouse standard technical specifications. This process, known as the Technical Specification Improvement Program (TSIP), is anticipated to remove many of the inconsistencies that currently exist in TS and provide uniform criteria for the industry. Therefore, in anticipation of the TSIP, the staff is currently deferring plant specific modifications that are scheduled to be included in the TSIP. The TSIP is currently scheduled to be complete later this year.

ENVIRONMENTAL CONSIDERATION

The amendment involves a change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposures. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set

forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 28, 1990

Principal Contributors: Douglas V. Pickett
Richard L. Emch
Carl S. Schulten