Docket No. 50-482

Mr. Bart D. Withers President and Chief Executive Officer Wolf Creek Nuclear Operating Corporation Post Office Box 411 Burlington, Kansas 66839 DISTRIBUTION: ADR4 Docket File GHill (4) NRC PDR Wanda Jones Local PDR JCalvo. PD4 Reading ACRS (10) **PNoonan** DPickett (2) GPA/PA OC/LFMB FHebdon DHagan OGC-Rockville Plant File EJordan

Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. 76099)

The Commission has issued the enclosed Amendment No. 39 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your

application dated March 2, 1990 (ET 90-0043).

The amendment revises the Technical Specification Table of Contents and Technical Specification Surveillance Requirement 4.8.4.1 to delete references to Table 3.8-1 which was previously removed from Technical Specifications in Amendment 28.

A copy of our related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Douglas V. Pickett, Project Manager Project Directorate IV Division of Reactor Projects - III, IV, V and Special Projects Office of Nuclear Reactor Regulation

Fnclosures:

 Amendment No. 39 to License No. NPF-42

2. Safety Evaluation

cc w/enclosures: See next page

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# NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

May 30, 1990

Docket No. 50-482

Mr. Bart D. Withers
President and Chief Executive Officer
Wolf Creek Nuclear Operating Corporation
Post Office Box 411
Burlington, Kansas 66839

Dear Mr. Withers:

SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO.39 TO FACILITY

OPERATING LICENSE NO. NPF-42 (TAC NO. 76099)

The Commission has issued the enclosed Amendment No.39 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station. The amendment consists of changes to the Technical Specifications in response to your application dated March 2, 1990 (ET 90-0043).

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A copy of our related Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Douglas V Pickett

Douglas V. Pickett, Project Manager Project Directorate IV Division of Reactor Projects - III, IV, V and Special Projects Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.39 to License No. NPF-42

2. Safety Evaluation

cc w/enclosures: See next page Mr. Bart D. Withers Wolf Creek Nuclear Operating Corporation

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# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

#### WOLF CREEK NUCLEAR OPERATING CORPORATION

#### WOLF CREEK GENERATING STATION

DOCKET NO. 50-482

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39 License No. NPF-42

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Wolf Creek Generating Station (the facility) Facility Operating License No. NPF-42 filed by the Wolf Creek Nuclear Operating Corporation (the Corporation), dated March 2, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-42 is hereby amended to read as follows:

#### 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 39, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The Corporation shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Frederick J. Hendon, Director

Project Directorate IV

Division of Reactor Projects - III,

IV, V and Special Projects

Office of Nuclear Reactor Regulation

Attachment: Charges to the Technical Specifications

Date of Issuance: May 30, 1990

# ATTACHMENT TO LICENSE AMENDMENT NO. 39

### FACILITY OPERATING LICENSE NO. NPF-42

#### DOCKET NO. 50-482

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE PAGES	INSERT PAGES	
XI	XI	
3/4 8-16	3/4 8-16	

# LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

SECTION		PAGE
PLANT SY	STEMS (Continued)	
3/4.7.10	DELETED	
TABLE 3.	7-3 DELETED	
3/4.7.11	DELETED	
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	7-4 AREA TEMPERATURE MONITORING	
3/4.8 E	LECTRICAL POWER SYSTEMS	
3/4.8.1	A.C. SOURCES	
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3/4.8.2	D.C. SOURCES	
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3/4.8.3	ONSITE POWER DISTRIBUTION	
	Operating	3/4 8-13
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3/4.8.4	ELECTRICAL EQUIPMENT PROTECTIVE DEVICES	
	Containment Penetration Conductor Overcurrent Protective Devices	3/4 0.30
		3/4 8-16

# LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

SECTION		PAGE
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3/4.9.1	BORON CONCENTRATION	3/4 9-1
3/4.9.2	INSTRUMENTATION	3/4 9-2
3/4.9.3	DECAY TIME	3/4 9-3
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3/4.9.8	RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION	
	High Water Level	
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3/4.9.12	SPENT FUEL ASSEMBLY STORAGE	3/4 9-15
FIGURE 3.	9-1 MINIMUM REQUIRED FUEL ASSEMBLY EXPOSURE AS A	
	FUNCTION OF INITIAL ENRICHMENT TO PERMIT STORAGE IN REGION 2	3/4 9-16
3/4.9.13	EMERGENCY EXHAUST SYSTEM	2/4 2 2

### ELECTRICAL POWER SYSTEMS

## ONSITE POWER DISTRIBUTION

#### SHUTDOWN

## LIMITING CONDITION FOR OPERATION

- 3.8.3.2 As a minimum, one of the following divisions of electrical busses shall be energized in the specified manner:
  - a. Division 1, consisting of:
    - 1) 4160-Volt Emergency Bus #NBOl. and
    - 2) 480-Volt Emergency Busses #NG01, NG03 and NG05E, and
    - 3) 120-Volt A.C. Vital Busses #NNO1 and NNO3 energized from their associated inverter connected to D.C. Busses #NKO1 and NKO3, and
    - 4) 125-volt D.C. Busses #NKOl and NKO3 energized from Batteries #NK11 and NK13 and Chargers #NK21 and NK23, or
  - b. Division 2, consisting of:
    - 1) 4160-Volt Emergency Bus #NB02, and
    - 2) 480-Volt Emergency Busses #NG02, NG04 and NG05E, and
    - 3) 120-Volt A.C. Vital Busses #NNO2 and NNO4 energized from their associated inverter connected to D.C. Busses #NKO2 and NKO4, and
    - 4) 125-Volt D.C. Busses #NKO2 and NKO4 energized from Batteries #NK12 and NK14 and Chargers #NK22 and #NK24.

APPLICABILITY: MODES 5 and 6.

#### ACTION:

Without one of the above required divisions of electrical busses energized in the required manner, immediately suspend all operations involving CORE ALTERATIONS, positive reactivity changes, or movement of irradiated fuel; initiate corrective action to energize at least one division of the required busses in the specified manner.

#### SURVEILLANCE REQUIREMENTS

4.8.3.2 The specified busses shall be determined energized in the required manner at least once per 7 days by verifying correct breaker alignment and indicated voltage on the busses.

## **ELECTRICAL POWER SYSTEMS**

# 3/4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

# CONTAINMENT PENETRATION CONDUCTOR OVERCURRENT PROTECTIVE DEVICES

## LIMITING CONDITION FOR OPERATION

3.8.4.1 For each containment penetration provided with a penetration conductor overcurrent protective device(s), each device shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### **ACTION:**

With one or more of the above required containment penetration conductor overcurrent protective device(s) inoperable:

- a. Restore the protective device(s) to OPERABLE status or deenergize the circuit(s) by tripping the associated backup circuit breaker or racking out or removing the inoperable circuit breaker within 72 hours, declare the affected system or component inoperable, and verify the backup circuit breaker to be tripped or the inoperable circuit breaker racked out, or removed, at least once per 7 days thereafter; the provisions of Specification 3.0.4 are not applicable to overcurrent devices in circuits which have their backup circuit breakers tripped, their inoperable circuit breakers racked out, or removed, or
- b. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

- 4.8.4.1 Protective devices required to be OPERABLE as containment penetration conductor overcurrent protective devices shall be demonstrated OPERABLE.
  - a. At least once per 18 months:
    - 1) By verifying that the 13.8 kV circuit breakers are OPERABLE by selecting, on a rotating basis, at least 10% of the circuit breakers, and performing the following:
      - a) A CHANNEL CALIBRATION of the associated protective relays,
      - b) An integrated system functional test which includes simulated automatic actuation of the system and verifying that each relay and associated circuit breakers and control circuits function as designed, and



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-42

### WOLF CREEK NUCLEAR OPERATING CORPORATION

#### WOLF CREEK GENERATING STATION

### **DOCKET NO. 50-482**

#### INTRODUCTION

Technical Specification 3/4.8.4 provides the requirements for Containment Penetration Conductor Overcurrent Protective Devices. On March 23, 1989, the NRC issued Amendment 28 which revised Technical Specification 3/4.8.4 and deleted Table 3.8-1, which listed the Containment Penetration Conductor Overcurrent Protective Devices. During a review of Amendment 28 by Wolf Creek Nuclear Operating Corporation personnel, it was discovered that there were still references to Table 3.8-1 in the Technical Specifications.

By letter dated March 2, 1990, the licensee proposed Technical Specification changes that would delete references to Table 3.8-1 in the Wolf Creek Generating Station (WCGS) Technical Specifications. These remaining references are located in the Table of Contents and in Surveillance Requirement 4.8.4.1.

#### **EVALUATION**

Table 3.8-1 provided information on the location, size and equipment protected by the Containment Penetration Conductor Overcurrent Protective Devices. The information which was in Table 3.8-1 is currently being administratively controlled in plant procedures. The references to this table which remain were inadvertently overlooked when the table was originally requested to be removed from the Technical Specifications. Removing the references to the table is now being pursued in order to eliminate any confusion resulting from referring to a table which no longer exists. This is an administrative change which does not change any of the requirements of the WCGS Technical Specifications. This change does not include any technical changes and is considered to be editorial in nature.

Based on the above discussions the staff is in agreement with the licensee's findings and, therefore, finds the proposed changes acceptable.

#### ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, and administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

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#### CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 30, 1990

Principal Contributor: D. Pickett