

Docket No.: 50482

Mr. Glenn L. Koester
Vice President - Nuclear
Kansas Gas and Electric Company
201 North Market Street
P. O. Box 208
Wichita, Kansas 67201

NOV 19 1986

Dear Mr. Koester:

Enclosed for your information is a "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing" related to your November 7, 1986, request concerning Technical Specifications 3.5.1.a and 3.5.2.e. The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

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Paul W. O'Connor, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

Enclosure: As stated

cc w/enclosure:
See next page

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Mr. Glenn L. Koester
Kansas Gas and Electric Company

Wolf Creek Generating Station
Unit No. 1

cc:

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November 19, 1986

DOCKET NO.

MEMORANDUM FOR: Rules and Procedures Branch
Division of Rules and Records
Office of Administration

FROM: Office of Nuclear Reactor Regulation

SUBJECT: Wolf Creek Generating Station (Kansas Gas and Electric Company, et al)

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (5) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Order.
- Exemption.
- Notice of Granting Exemption.
- Environmental Assessment.
- Notice of Preparation of Environmental Assessment.
- Other: _____

NOTE: Please insert a date (30 days from publication) in second paragraph of page 6. Please call me with date. Thanks.

Office of Nuclear Reactor Regulation

Enclosure:
As stated

Contact: Marilee Duncan
Phone: Ext. 28928

OFFICE	PAD#4						
SURNAME	MDuncan						
DATE	11/19/86						

UNITED STATES NUCLEAR REGULATORY COMMISSIONKANSAS GAS AND ELECTRIC COMPANYKANSAS CITY POWER AND LIGHT COMPANYKANSAS ELECTRIC POWER COOPERATIVE, INC.DOCKET NO. 50-482NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-42, issued to Kansas Gas and Electric Company, Kansas City Power and Light Company, and Kansas Electric Power Cooperative, Inc. (the licensee), for operation of the Wolf Creek Generating Station located in Coffey County, Kansas. These changes were requested in the licensee's letter dated November 7, 1986.

The proposed amendment would change Technical Specification 3.5.1.a to allow closure of one ECCS accumulator isolation valve in MODE 3 above 1000 psig during startup, while performing Surveillance Requirement 4.4.6.2.2. This will only be done providing RHR pump discharge valves EJ HV-8809A and/or B are not closed and the closed isolation valve is capable of being reopened.

Also, the proposed amendment would change Technical Specification 3.5.2.e to allow closure of EJ HV-8809A and/or B in MODE 3 during startup, while performing Surveillance Requirement 4.4.6.2.2. This will only be done providing the closed valve(s) is (are) capable of being reopened and pressurizer pressure is below 1000 psig, and if above 100 psig, no ECCS accumulator isolation valve is closed.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

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The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

In accordance with the requirements of 10 CFR 50.92, the licensee submitted the following significant hazards determination: The proposed change to Technical Specification 3.5.1 does not involve a significant hazards consideration because operation of Wolf Creek Generating Station in accordance with this change would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated. The maximum credible LOCA to be considered for the RCS pressure boundary during shutdown operation would be a 6 inch pipe break. It has been determined that low pressure shutdown and startup operating conditions are so far below the conditions for which the Reactor Coolant System (RCS) Loss of Coolant Accident (LOCA) is not credible that (and) for all practical purposes this accident can be assumed not to occur. For this credible LOCA, the RCS break flow rate and depressurization rate are significantly less than for a design basis large break LOCA. For startup conditions, however, the break flow and depressurization rates would be further reduced due to the lower initial RCS pressure and temperature. In addition, the initial fuel rod temperature and decay heat level would be significantly less than for full power since the reactor would have been shutdown for a period of time. With this

longer depressurization time and lower decay levels, there is ample time available for operator action to open the closed accumulator valve.

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated. This Technical Specification change pertains to LOCAs and to how much Emergency Core Cooling System (ECCS) flow is available immediately and after operator action. The possible slight delay in initiating full ECCS flow does not affect any other kind of accident.

(3) Involve a significant reduction in a margin of safety. The 10 CFR 50.59 Safety Evaluation for this Technical Specification amendment has concluded that the maximum credible LOCA during heatup is the rupture of a 6 inch pipe. During the period of time when this valve may be closed per the proposed Amendment, RCS pressure will be above the pressure at which the accumulators can inject water. Therefore it is concluded that during the depressurization of the RCS following a LOCA, the operators will recognize the condition and will be able to reopen the closed accumulator valve and prevent any significant fuel heatup. The valve will be able to be reopened from the main control room during this period.

The proposed change to Technical Specification 3.5.2 does not involve a significant hazards consideration because operation of Wolf Creek Generating Station in accordance with this change would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated. It has been determined that low pressure shutdown and startup operating conditions are so far below the conditions for which the RCS has been designed, that a large LOCA is not credible and for all practical purposes can be assumed not to occur. It has been concluded that

the maximum credible LOCA to be considered for the RCS pressure boundary during shutdown operation would be a 6 inch pipe break. For a credible LOCA, the RCS break flow rate and depressurization rate is significantly less than for a design basis large break LOCA. For startup conditions, the break flow and depressurization rates would be further reduced due to the lower initial RCS pressure and temperature. In addition, the initial fuel rod temperature and decay heat level would be significantly less than for full power since the reactor would have been shutdown for a period of time. With this longer depressurization time and lower decay heat levels, there is ample time available for operator action to open the closed RHR valve or valves.

(2) Create the possibility of a new or different kind of accident from any accident previously evaluated. This Technical Specification change pertains to LOCAs and to how much ECCS flow is available immediately and after operator action. The possible slight delay in initiating full ECCS flow does not affect any other kind of accident.

(3) Involve a significant reduction in a margin of safety. The 10 CFR 50.59 Safety Evaluation for this Technical Specification amendment has concluded that the maximum credible LOCA during heatup is the rupture of a 6 inch pipe. During the period of time when EJ HV-8809A and/or B may be closed per the proposed Amendment, RCS pressure will be above the pressure at which the RHR pumps can inject water. Therefore it is concluded the during the depressurization of the RCS following a LOCA, the operators will recognize the condition and will be able to reopen the respective closed valve(s) and prevent any significant fuel heatup. The valve(s) will be able to be reopened from the main control room during this period.

During startup, the low pressure safety injection signal is blocked until RCS pressure exceeds 1970 psig. Therefore should a LOCA occur below 1970 psig, operator action would be required to initiate any ECCS flow. When this occurs, the operator will also open any valves that had been closed. If above 1970 psig, the safety injection signal will be unblocked. Should a LOCA occur, the safety injection signal will start both centrifugal charging pumps, both safety injection pumps, both RHR pumps and if any accumulator valve is closed, it will automatically open it. Since the RHR pumps cannot inject water until RCS pressure drops to approximately 190 psig, two charging pumps, two safety injection pumps and four accumulators will be injecting or will have injected into the core before the RCS has depressurized to 190 psig. This allows adequate time for the operator to open EJ HV-8809A and/or B, before the pumps are needed to inject.

Based on the above analysis, the licensee concluded that the proposed amendment does not involve significant hazards considerations. The staff has reviewed the licensee's significant hazards consideration determination and agrees with the licensee's analysis. The staff has, therefore, made a proposed determination that the licensee's request does not involve a significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555. Comments may also be delivered to Room 4000, Maryland National Bank Building, Bethesda, Maryland from 8:15 a.m. to 5:00 p.m. Monday through Friday. Copies of comments received may be examined at the NRC Public Document Room, 1717 H Street, N. W., Washington, D. C.

By _____, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding;

(2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspects(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitation in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination of the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room 1717 H Street, N. W., Washington, D. C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union

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operator should be given Datagram Identification Number 3737 and the following messaged addressed to B. J. Youngblood: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, and to Jay Silberg, Esq., Shaw Pittman, Potts and Trowbridge, 1800 M Street, N.W., Washington, D.C. 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Emporia State University, William Allen White Library, 1200 Commercial Street, Emporia, Kansas and the Washburn University School of Law Library, Topeka, Kansas.

Dated at Bethesda, Maryland, this 19th day of November 1986.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A, NRR

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