

May 15, 2003

Mr. Mark E. Warner, Site Vice President
c/o Mr. James M. Peschel
Seabrook Station
P.O. Box 300
Seabrook, NH 03874

SUBJECT: SEABROOK STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT
RE: MISSED SURVEILLANCES USING CONSOLIDATED LINE ITEM
IMPROVEMENT PROCESS (TAC NO. MB4749)

Dear Mr. Warner:

The Commission has issued the enclosed Amendment No. 87 to Facility Operating License No. NPF-86 for the Seabrook Station, Unit No 1 (Seabrook Station), in response to the application dated March 22, 2002, as supplemented May 13, June 24, and July 29, 2002, filed by North Atlantic Energy Service Corporation (NAESCO) as the then licensee for Seabrook Station. On November 1, 2002, the U.S. Nuclear Regulatory Commission (NRC or Commission) approved the transfer of the license for Seabrook Station, to the extent held by NAESCO, and certain co-owners of the facility, on whose behalf NAESCO was also acting, to FPL Energy Seabrook, LLC (FPLE Seabrook). By letter dated December 20, 2002, FPLE Seabrook requested that the NRC continue to review and act upon all requests before the Commission that had been submitted by NAESCO. By another letter, also dated December 20, 2002, FPLE Seabrook, submitted a supplement to the application. Accordingly, we have completed our review of the March 22, 2002, application.

The amendment revises Technical Specification (TS) Surveillance Requirement (SR) 4.0.1 to clarify the definition and treatment of missed surveillances. SR 4.0.3 is revised to extend the delay period, before entering a Limiting Condition for Operation, following a missed surveillance. The delay period is extended from the current limit of "... up to 24 hours" to "...up to 24 hours or up to the limit of the specified surveillance interval, whichever is greater." In addition, the following requirement is added to SR 4.0.3: "A risk evaluation shall be performed for any surveillance delayed greater than 24 hours and the risk impact shall be managed." The amendment includes a TS requirement for a TS Bases Control Program and also makes administrative changes to SRs 4.0.1 and 4.0.3 to be consistent with NUREG-1431, Revision 2, "Standard Technical Specifications, Westinghouse Plants."

M. Warner

- 2 -

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 87 to NPF-86
2. Safety Evaluation

cc w/encls: See next page

M. Warner

- 2 -

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosures: 1. Amendment No. 87 to NPF-86
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION:

PUBLIC	VNerses	GHill(2)
PDI-2 Rdg.	CRaynor	RDennig
SRichards	OGC	DJaffe
JClifford	ACRS	BMcDermott, RGN-I

ADAMS Accession Numbers: ML022000461, TS(s): ML , Package: ML

*See previous concurrence.

OFFICE	PDI-2/PM	PDI-2/LA	OGC*	RORP/SC*	PDIV-1/PM*	PDI-2/SC
NAME	VNerses	CRaynor	RWeisman	RDennig	DJaffe	JClifford
DATE	5/8/03	5/7/03	3/3/03	1/21/03	1/21/03	5/14/03

OFFICIAL RECORD COPY

Seabrook Station, Unit No. 1

cc:

Mr. J. A. Stall
Senior Vice President, Nuclear and
Chief Nuclear Officer
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Mr. Peter Brann
Assistant Attorney General
State House, Station #6
Augusta, ME 04333

Resident Inspector
U.S. Nuclear Regulatory Commission
Seabrook Nuclear Power Station
P.O. Box 1149
Seabrook, NH 03874

Town of Exeter
10 Front Street
Exeter, NH 03823

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Office of the Attorney General
One Ashburton Place
20th Floor
Boston, MA 02108

Board of Selectmen
Town of Amesbury
Town Hall
Amesbury, MA 01913

Mr. Dan McElhinney
Federal Emergency Management Agency
Region I
J.W. McCormack P.O. &
Courthouse Building, Room 401
Boston, MA 02109

Mr. Stephen McGrail, Director
ATTN: James Muckerheide
Massachusetts Emergency Management
Agency
400 Worcester Road
Framingham, MA 01702-5399

Philip T. McLaughlin, Attorney General
Steven M. Houran, Deputy Attorney
General
33 Capitol Street
Concord, NH 03301

Mr. Donald Bliss, Director
New Hampshire Office of Emergency
Management
State Office Park South
107 Pleasant Street
Concord, NH 03301

Mr. Daniel G. Roy
Nuclear Training Manager
Seabrook Station
FPL Energy Seabrook, LLC
P.O. Box 300
Seabrook, NH 03874

Mr. James M. Peschel
Regulatory Programs Manager
Seabrook Station
FPL Energy Seabrook, LLC
P.O. Box 300
Seabrook, NH 03874

Mr. Gene F. St. Pierre
Station Director
Seabrook Station
FPL Energy Seabrook, LLC
P.O. Box 300
Seabrook, NH 03874

Mr. M. S. Ross, Attorney
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Seabrook Station, Unit No. 1

cc: continued

Mr. Rajiv S. Kundalkar
Vice President - Nuclear Engineering
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

FPL ENERGY SEABROOK, LLC, ET AL.*

DOCKET NO. 50-443

SEABROOK STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87
License No. NPF-86

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated March 22, 2002, as supplemented May 13, June 24, July 29, and December 20, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*FPL Energy Seabrook, LLC (FPLE Seabrook), is authorized to act as agent for: the Hudson Light & Power Department, Massachusetts Municipal Wholesale Electric Company, and Taunton Municipal Light Plant, and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-86 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 87, and the Environmental Protection Plan contained in Appendix B are incorporated into Facility License No. NPF-86. FPLE Seabrook shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 15, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. NPF-86

DOCKET NO. 50-443

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
3/4 0-2	3/4 0-2
6-14D	6-14D
B 3/4 0-4	B 3/4 0-4
B 3/4 0-5	B 3/4 0-5
B 3/4 0-6	B 3/4 0-6
- - -	B 3/4 0-7

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. NPF-86

FPL ENERGY SEABROOK, LLC

SEABROOK STATION, UNIT NO. 1

DOCKET NO. 50-443

1.0 INTRODUCTION

By application dated March 22, 2002, as supplemented May 13, June 24, and July 29, 2002, the North Atlantic Energy Service Corporation (NAESCO), as the then licensee for Seabrook Station, Unit No. 1 (Seabrook Station), submitted a request for changes to the Seabrook Station, Technical Specifications (TSs). On November 1, 2002, the U.S. Nuclear Regulatory Commission (NRC or the Commission) approved the transfer of the license for Seabrook Station, to the extent held by NAESCO and certain co-owners of the facility on whose behalf NAESCO was also acting, to FPL Energy Seabrook, LLC (FPLE Seabrook). By letters dated December 20, 2002, FPLE Seabrook supplemented the application and requested that the NRC continue to review and act upon all requests before the Commission that had been submitted by NAESCO. Accordingly, the NRC staff has completed a review of the March 22, 2002, application. The NRC staff's findings are presented below.

The proposed amendment would revise TS Surveillance Requirement (SR) 4.0.1 to clarify the definition and treatment of missed surveillances. SR 4.0.3 would be updated to extend the delay period, before entering a Limiting Condition for Operation (LCO), following a missed surveillance. The delay period would be extended from the current limit of "... up to 24 hours" to "...up to 24 hours or up to the limit of the specified surveillance interval, whichever is greater." In addition, the following requirement would be added to SR 4.0.3: "A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed." The proposed amendment would also include a TS Bases Control Program and would make administrative changes to SRs 4.0.1 and 4.0.3 to be consistent with NUREG-1431, Revision 2, "Standard Technical Specifications, Westinghouse Plants." The Bases for TSs 4.0.1 and 4.0.3 would also be revised consistent with the proposed changes.

2.0 BACKGROUND

The licensee's proposal follows one of the industry's initiatives under the risk-informed TS program. The licensee's application references TS Task Force (TSTF)-358, Revision 6, which incorporates changes to TSTF-358, Revision 5, made in response to a notice published in the *Federal Register* (FR) on June 14, 2001 (66 FR 32400), seeking public comment. With respect to variations or deviations from the TS changes in TSTF-358, Revision 6, the licensee stated in its application that administrative changes to SRs 4.0.1 and 4.0.3 are also required to make the current Seabrook Station TSs consistent with NUREG-1431, Revision 2. These changes are

necessary in order to make the current Seabrook Station TSs compatible with the proposed changes in TSTF-358. The licensee also stated in its application that it reviewed the staff's model safety evaluation (SE) dated June 14, 2001, as modified by the comments and responses published in the FR on September 28, 2001 (66 FR 49714), and concluded that the justifications presented in the TSTF proposal and the model SE are applicable to Seabrook Station.

In a letter dated November 17, 1999, the Nuclear Energy Institute (NEI) TSTF proposed several changes to the improved Standard Technical Specifications (STS) (i.e., NUREGs 1430 - 1434) on behalf of the industry. One of the proposed changes, identified as TSTF-358, was a change to STS SR 3.0.3 regarding missed SRs. On February 14, 2000, the staff requested that the NEI TSTF modify TSTF-358 to address several questions and comments that the staff had during their initial review of the proposed change. On September 15, 2000, the NEI TSTF submitted Revision 5 to TSTF-358 for review. Revisions 2 through 4 were only reviewed by the industry and were never submitted for NRC review. In response to comments resulting from the request for public comments in the FR notice (66 FR 32400) of June 14, 2001, the NEI TSTF submitted Revision 6 to TSTF-358 for review on September 14, 2001, and it was approved by the NRC on October 1, 2001.

Parts of the SE contained herein was published in the FR on June 14, 2001 (66 FR 32400). The NRC staff has since made minor, editorial changes to the SE. In addition, this SE includes a plant-specific evaluation of the Seabrook Station application, since Seabrook Station has not adopted the STS.

The regulations contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical Specifications," require that TSs include SRs. SRs are requirements relating to test, calibration, or inspection to ensure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met. The TSs require surveillance tests to be performed periodically (e.g., weekly or monthly). The periodic test interval defined in the TSs is called the surveillance frequency or surveillance interval. The majority of surveillance tests included in the TSs are designed to ensure that standby safety systems will be operable when they are needed to mitigate an accident. By testing these components, failures that may have occurred since the previous test can be detected and corrected.

STS SR 3.0.1 states that SRs shall be met during the MODES or other specified conditions in the applicability for individual LCOs, and that failure to perform a surveillance within the specified frequency shall be a failure to meet the LCO, except as provided in SR 3.0.3.

The current STS SR 3.0.3 requires that, if it is found that a surveillance test was not performed within its specified frequency, the associated LCO be declared not met (e.g., equipment be declared inoperable) unless the missed surveillance test is completed successfully within 24 hours or within the limit of the specified frequency, whichever is less, from the time it was discovered that the test was not performed. The requirements in STS SR 3.0.3 are based on NRC Generic Letter (GL) 87-09, "Sections 3.0 and 4.0 of the Standard Technical Specification (STS) of the Applicability of Limiting Conditions for Operation and Surveillance Requirements," dated June 4, 1987.

GL 87-09 was published to address three specific issues with the application of TSs. One of those issues is missed surveillances. The GL states: “The second problem involves unnecessary shutdowns caused by Specification 4.0.3 when surveillance intervals are inadvertently exceeded. The solution is to clarify the applicability of the Action Requirements, to specify a specific acceptable time limit for completing a missed surveillance in certain circumstances, and to clarify when a missed surveillance constitutes a violation of the Operability Requirements of an LCO. It is overly conservative to assume that systems or components are inoperable when a surveillance has not been performed, because the vast majority of surveillances do, in fact, demonstrate that systems or components are OPERABLE. When a surveillance is missed, it is primarily a question of operability that has not been verified by the performance of a Surveillance Requirement. Because the allowable outage time limits of some Action Requirements do not provide an appropriate time for performing a missed surveillance before Shutdown Requirements apply, the TS[s] should include a time limit that allows a delay of required actions to permit the performance of the missed surveillance based on consideration of plant conditions, adequate planning, availability of personnel, the time required to perform the surveillance, and, of course, the safety significance of the delay in completing the surveillance. The staff has concluded that 24 hours is an acceptable time limit for completing a missed surveillance when the allowable outage times of the Action Requirements are less than this limit, or when time is needed to obtain a temporary waiver¹ of the Surveillance Requirement.” [emphasis added]

The proposed change would extend the delay time for declaring the LCO not met and entering the required actions by allowing more time to perform the missed surveillance test. This would be achieved by modifying SR 3.0.3 to allow a delay period from 24 hours up to the surveillance frequency, whichever is greater, to perform a missed surveillance prior to having to declare the LCO not met. The proposed change would also add a sentence to SR 3.0.3 that states, “A risk evaluation shall be performed for any surveillance delayed greater than 24 hours, and the risk impact shall be managed.”

The objective of the proposed change is to minimize the impact on plant risk resulting from the performance of a missed surveillance test by allowing flexibility in considering the plant conditions and other plant activities without compromising plant safety. In addition, implementation of the proposed change would reduce the need for the licensee to apply for regulatory relief to delay the performance of missed surveillances.

The basis for establishing the changes to requirements for missed surveillances in GL 87-09 continues to apply to the current proposed change to SR 3.0.3. As evidenced by the discussion in GL 87-09, the intent of the change proposed in the GL was to reduce the impact on plant risk resulting from the performance of a missed surveillance test by allowing some flexibility in the performance of missed tests. The delay time of 24 hours was selected using engineering judgement in the absence of suitable tools to determine a delay period on a case-by-case basis. In addition, the staff recognized in GL 87-09 that even a 24-hour delay period would not be sufficient in some cases, and licensees would need to seek regulatory relief in those cases.

¹The terminology “temporary waiver” was subsequently revised to refer to the practice as “enforcement discretion.”

The recent revision to the Maintenance Rule to establish the requirement in 10 CFR 50.65(a)(4) to assess and manage the increase in risk that may result from maintenance activities provides a framework to allow a more risk-informed approach to addressing missed surveillances. This approach is consistent with the Commission's policy to increase the use of probabilistic risk assessment (PRA) technology in all regulatory matters to the extent supported by the state-of-the-art PRA methods and data, and continues to support the objectives outlined by the staff in GL 87-09.

The NRC staff believes that the proposed change to SR 3.0.3 is appropriate because: (1) the number of missed surveillance tests is a very small fraction of the total number of such tests performed at a nuclear plant each year; (2) the change applies to unintentionally missed surveillance tests and is not intended to be used as an operational convenience to extend surveillance frequencies (as stated in the existing SR 3.0.3 Bases); and (3) missed surveillances will be placed in the licensee's corrective action program (CAP).

The NRC staff has determined that the proposed change is applicable to all licensees. In GL 87-09, the staff concluded that the proposed modifications would result in improved TSs for all plants and no limitations were put on the applicability of the proposed changes. Because the basis for this proposed change is largely the same as for the change proposed in GL 87-09, the staff believes the same broad applicability is appropriate. In addition, every licensee is required to comply with the Maintenance Rule and, therefore, will have implemented programs to comply with 10 CFR 50.65(a)(4) to assess and manage risk associated with maintenance and other operational activities.

The guidance outlining an acceptable process for licensees to assess and manage increases in risk that may result from planned maintenance activities is found in Regulatory Guide (RG) 1.182. RG 1.182 endorses a revised Section 11 to NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," Revision 2, dated February 22, 2000, updated by NEI.

Section 11 of NUMARC 93-01 provides guidance for assessing and managing risk impact resulting from performance of maintenance activities, including guidance for establishing action thresholds based on qualitative and quantitative considerations as well as risk management actions. The objective of risk management is to control the temporary and aggregate risk increases from maintenance activities such that the plant's average baseline risk is maintained within a minimal range. This is accomplished by using the results of the risk assessment to plan and schedule maintenance such that the risk increases are limited, and to take additional actions beyond routine work controls to address situations where the temporary risk increase is above a certain threshold.

In order to gain additional insights into the proposed change, the NRC staff referred to the regulatory guidance provided in RG 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," dated July 1998 and in RG 1.177, "An Approach for Plant-Specific, Risk-Informed Decision making: Technical Specifications," dated August 1998, although these RGs do not specifically address the type of change in this proposal. RG 1.177 provides the NRC staff's recommendations for utilizing risk information to evaluate changes to nuclear power plant TSs by assessing the impact of such proposed changes on the risk associated with plant operation. The approach documented in RG 1.177 was taken into consideration by the NRC staff in

evaluating the risk information provided in support of the proposed changes in SR 4.0.3 to increase the time allowed to perform a missed surveillance.

One portion of the guidance in RG 1.177 includes the assessment of the risk impact of the proposed change for comparison to acceptance guidelines consistent with the Commission's Safety Goal Policy Statement, as documented in RG 1.174. In addition, the approach outlined in the guidance aims at ensuring that the plant risk does not increase unacceptably at any time during the implementation of the proposed change (i.e., during the extended surveillance interval).

Another portion of the guidance addresses the need for identifying risk-significant configurations resulting from maintenance or other operational activities and taking appropriate compensatory measures to avoid such configurations. This type of evaluation is directly addressed by the requirement to perform a risk assessment for missed surveillances delayed longer than 24 hours.

The NRC staff believes that insights from the guidance provided in RGs 1.174 and 1.177 can be used to show how the proposed change is expected to result in, at most, an increase in risk which is small and consistent with the Commission's Safety Goal Policy Statement. The NRC staff believes that, in the majority of the cases of missed surveillances, implementation of the proposed change will result in a risk benefit due to the proposed requirement for the licensee to evaluate the risk impact for missed surveillances that would require a delay of longer than 24 hours.

3.0 EVALUATION

As discussed in the licensee's application, incorporation of the changes in TSTF-358, Revision 6, necessitates three proposed changes to the Seabrook Station TSs: (1) editorial changes to SRs 4.0.1 and 4.0.3 and the associated TS Bases to be consistent with NUREG-1431, Revision 2; (2) incorporation of a Bases Control Program in TS Section 6.0; and (3) changes to SRs 4.0.1, 4.0.3, and the associated TS Bases to incorporate TSTF-358, Revision 6. The three proposed changes are addressed in the following evaluation.

3.1 Editorial Changes to SRs 4.0.1 and 4.0.3

The licensee has proposed to revise Seabrook Station SRs 4.0.1 and 4.0.3 and the associated TS Bases to be consistent with NUREG-1431, Revision 2, SRs 3.0.1 and 3.0.3. These changes are necessary to make the current Seabrook Station TSs compatible with the proposed changes to TSTF-358, Revision 6. The NRC staff has reviewed the proposed changes and finds that they include minor editorial changes and changes in terminology (e.g., use of "surveillance interval" instead of "Frequency"), and are consistent with NUREG-1431, Revision 2.

The staff concludes that the proposed editorial changes and changes in terminology are editorial in nature, provide further clarity to the Seabrook TSs, and have no impact on safety. Therefore, the proposed changes are acceptable. The proposed changes to SRs 4.0.1 and 4.0.3 needed to implement TSTF-358, Revision 6, are discussed in Section 3.3 of this SE.

3.2 Incorporation of a Bases Control Program

The FR Notice dated September 28, 2001 (66 FR 49714), "Notice of Availability of Model Application Concerning Technical Specification Improvement To Modify Requirements Regarding Missed Surveillances Using the Consolidated Line Item Improvement Process," stated that the proposed change to modify the TS requirements for missed surveillances is applicable to all licensees who currently have, or who will adopt, in conjunction with the proposed change, TS requirements for a Bases control program consistent with the TS Bases Control Program described in Section 5.5 of the applicable vendor's STS.

Prior to the issuance of the STS (NUREGS 1430-1434), the control of TS Bases was not clearly defined by either TSs or NRC regulations. The administrative requirements for a Bases control program were added to the STS to define a methodology for evaluating changes to and providing updates of the TS Bases. The addition of the TS Bases Control Program for plants that have not adopted the STS would provide the same benefits in terms of defining a methodology for the maintenance of the TS Bases.

Since the licensee proposed to incorporate administrative controls for a TS Bases control program that are consistent with the STS requirements, the applicability condition included in the FR Notice for the use of the Consolidated Line Item Improvement Process (CLIP) for the missed surveillances TS change is satisfied.

3.3 Incorporation of TSTF-358, Revision 6

The requirements pertaining to missed surveillances for Seabrook Station are contained in TS 4.0.1 and TS 4.0.3. The proposed change would revise SR 4.0.1 to explicitly state that failure to meet a surveillance constitutes a failure to meet the limiting condition for operation. Additionally the revised 4.0.1 states that, except as provided in SR 4.0.3, failure to perform a surveillance within the specified surveillance interval shall be considered failure to meet the LCO. In concert with this, the proposed revision to SR 4.0.3. would extend the delay period, following a missed surveillance, for entering a LCO. The delay period would be extended from the current limit of "... up to 24 hours" to "...up to 24 hours or up to the limit of the specified surveillance interval, whichever is greater." In addition, the following requirement would be added to SR 4.0.3: "A risk evaluation shall be performed for any surveillance delayed greater than 24 hours and the risk impact shall be managed."

The proposed change will not allow equipment known to be inoperable to be considered operable until the missed surveillance is performed. If it is known that the missed surveillance could not be met, SR 4.0.3 would require that the LCO be declared not met and the appropriate condition(s) entered. In addition, the Bases for SR 4.0.3 state that the use of the delay period established by SR 4.0.3 is a flexibility which is not intended to be used as an operational convenience to extend surveillance intervals, but only for the performance of missed surveillances.

The modification would also include changes to the Bases for SR 4.0.1 and SR 4.0.3 that provide details on how to implement the new requirements. The Bases changes provide guidance for surveillance frequencies that are not based on time intervals but are based on specified unit conditions, operating situations, or requirements of regulations. In addition, the Bases changes state that the licensee is expected to perform any missed surveillance test at

the first reasonable opportunity, taking into account appropriate considerations, such as the impact on plant risk and accident analysis assumptions, consideration of unit conditions, planning, availability of personnel, and the time required to perform the surveillance. The Bases also state that the risk impact should be managed through the program in place to implement 10 CFR 50.65(a)(4) and its implementation guidance in RG 1.182, "Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants," dated May 2000, and that the missed surveillance should be treated as an emergent condition as discussed in RG 1.182. In addition, the Bases state that the degree of depth and rigor of the evaluation should be commensurate with the importance of the component and that missed surveillances for important components should be analyzed quantitatively. The Bases also state that, if the results of the risk evaluation determine that the risk increase is significant, the evaluation should be used to determine the safest course of action. Finally, the Bases state that all missed surveillances will be placed in the licensee's CAP.

The proposed change to SR 4.0.1 clarifies what constitutes a failed surveillance. This change does not modify the scheduling or performance of surveillances and is, therefore, acceptable.

Key elements provided by the licensee to justify the proposed TS change to SR 4.0.3 are listed below. These elements were built into the process to ensure that every time a surveillance is missed, the risk will be properly assessed and managed. In addition, such elements facilitate regulatory oversight.

- A risk evaluation shall be performed for any surveillance test delayed longer than 24 hours and the risk impact shall be managed.
- Although the proposed change to SR 4.0.3 allows an increase of the delay time, the missed surveillance test should be performed at the "first reasonable opportunity."
- The "first reasonable opportunity" will be determined by taking into consideration the risk impact from delaying the surveillance test (including risk from changing plant configurations or shutting the plant down to perform the surveillance, whenever applicable), as well as the impact on any analysis assumptions, in addition to unit conditions, planning, availability of personnel, and the time required to perform the surveillance.
- A missed surveillance will be treated as an emergent condition in the same fashion as other unplanned maintenance activities. The risk impact of the condition will be managed through the program in place to implement 10 CFR 50.65(a)(4) and its implementation guidance, RG 1.182.
- A missed surveillance will be placed in the licensee's CAP, thus providing the NRC staff with a means to verify that the number of missed surveillances continues to be very low.
- The NRC's operating reactor oversight process (ROP) will provide the framework for inspectors and other NRC staff to review missed surveillances and assess the licensee's actions and performance.

The NRC staff finds that a process containing these key elements is appropriate in this case for the following reasons:

- Section 50.65 (a)(4) of 10 CFR requires licensees to implement programs to assess and manage increases in risk that may result from planned maintenance activities. This program is suitable to assess and manage the risk impact of missed surveillances because missed surveillances can be treated as emergent conditions and their risk impact will be assessed and managed in an integrated fashion with concurrent maintenance activities.
- Inspection procedures are in place which will allow NRC staff to oversee the implementation of Maintenance Rule requirements, including the adequacy of risk assessments performed by licensees for maintenance configurations.
- The number of missed surveillance tests is a very small fraction of the total number of such tests performed at a nuclear plant each year. The proposed change is not intended to be used as an operational convenience to extend surveillance frequencies.
- This process is similar to other improvements that have been made to the TSs that allow the use of a controlled decision-making process by licensees when the process has some high-level regulatory oversight. Two examples of this are the adoption of the Core Operating Limits Report, and the Pressure/Temperature Limits Report. In each of these cases, the NRC staff approved the methodology behind the calculation of certain TS parameter limits and then allowed the specific limits to be removed from TSs and controlled by the licensee using the approved methodology. Similarly, for this proposed change, the NRC staff has already approved guidance that outlines a process for complying with 10 CFR 50.65(a)(4) and, therefore, can allow the licensee to use that guidance to determine the most prudent course of action in the case of a missed surveillance.

3.4 Risk Impact of the Proposed Change

The NRC staff made a qualitative assessment of the risk impact of the proposed change for comparison with the intent of the acceptance guidelines documented in RG 1.174, consistent with the Commission's Safety Goal Policy Statement. Such risk impact is measured by the average (yearly) risk change. In addition, the NRC staff took into consideration guidance in RG 1.177 aimed at ensuring that the plant risk does not increase unacceptably at any time during the implementation of the proposed change (i.e., during an extended surveillance interval in this case). The NRC staff's qualitative assessment is summarized below.

3.4.1 Average Risk Impact

The probability that a standby active component, such as a pump or a circuit breaker, will fail when demanded during an accident is based on the assumption that the component fails due to "standby" stresses (i.e., stresses which are present while the component is in standby, such as

“corrosion, dirt, lack of lubrication). This probability, also called the component’s average unavailability,” is used in PRAs and is most frequently calculated by the following equation:

$$q = \frac{1}{2} * \lambda * T$$

where:

q = the component’s average unavailability,
λ = the component’s failure rate (assumed constant) while in standby, and
T = the interval at which the component is tested for operability.

The average unavailability of a structure, system, or component (SSC), calculated by using the above equation, reflects the potential vulnerability of the component to “standby” stresses. Such vulnerability increases with time between operability checks (tests) assuming corrective action is taken to restore failed components identified by the test. Thus, the risk impact of a missed surveillance is reflected by the increased unavailability of the related SSCs due to the increase of the interval between surveillance tests. If the missed surveillance affects two or more components, some “standby” stresses may impact multiple components. In such a case, the missed surveillance would also increase the average common cause failure (CCF) unavailability of two or more components and this should be addressed in the risk assessment. (CCF unavailabilities are calculated by adjusting the single component failure unavailability using standard PRA techniques, such as the beta factor or the Multiple Greek Letter method.)

The thresholds of the aggregate risk impacts are based on the permanent change guidelines discussed in RG 1.174. The licensee will be expected to manage the risk from the proposed TS change in conjunction with the risk from other concurrent plant activities to ensure that any risk increase, in terms of Core Damage Frequency (CDF) and Large Early Release Frequency (LERF), will be small and consistent with the Commission’s Safety Goal Policy Statement.

Risk insights from existing PRAs and the low frequency of missed surveillances indicate that the proposed TS change is highly unlikely to lead to a significant increase in the average (yearly) risk, in terms of CDF or LERF. Significant risk increases can occur only under the following conditions:

- The number of missed surveillances is allowed to increase significantly;
- High risk configurations are allowed (e.g., by allowing certain combinations of multiple missed surveillances and/or outages); and
- Poor risk management of plant operational activities is allowed.

Any of these conditions would be in violation of proposed SR 4.0.3 and could trigger a review by NRC of the licensee’s actions and performance. The implementation guidance found in the proposed SR 4.0.3 Bases is intended to ensure that such conditions would not occur. Also, the licensee is required to manage risk associated with online maintenance activities. Furthermore, the addition of missed surveillances (rather rare plant conditions) to the maintenance activities is not expected to increase risk. On the contrary, insights from existing risk assessments indicate that there are plant conditions during which it is preferable and safer not to have to

complete missed surveillance tests for some SSCs. Therefore, the proposed TS change will allow the licensee to make informed decisions and take appropriate actions to control risk.

3.4.2 Temporary risk impact

In addition to changes in the mean values of CDF and LERF, the incremental conditional core damage probability (ICCDP) and the incremental conditional large early release probability (ICLERP) are proposed by RG 1.177 as appropriate measures of the increase in probability of core damage and large early release, respectively, during the period of implementation of a proposed TS change (i.e., during the extended surveillance period in the case of a missed surveillance). RG 1.182 provides guidance for controlling temporary risk increases resulting from maintenance activities. Such guidance, which is consistent with guidance provided in RG 1.177, establishes action thresholds based on qualitative and quantitative considerations as well as risk-management actions. The NRC staff expects that the licensee will implement this guidance for assessing temporary risk increases from missed surveillances concurrently with maintenance and other operational activities.

Instantaneous and temporary risk increases from a missed surveillance are assessed by considering the time-dependent unavailability, most often calculated by the following equation:

$$q(t) = \lambda * t$$

where:

$q(t)$ = the component's unavailability at time t
 λ = the component's failure rate (assumed constant) while in standby, and
 t = time from end of surveillance frequency of a missed surveillance test.

If the missed surveillance affects two or more components, some "standby" stresses may impact multiple components. In such a case, the missed surveillance would also increase the time-dependent CCF unavailability of two or more components and this should be addressed in the risk assessment.

Significant temporary risk increases following a missed surveillance can occur only under the following conditions:

- High risk configurations are allowed (e.g., by allowing certain combinations of multiple missed surveillances and/or outages), and
- Poor risk management of plant operation activities is allowed.

Either of these conditions would be in violation of proposed SR 4.0.3 and could trigger an NRC review of the licensee's actions and performance. The requirements associated with the proposed change are intended to ensure that such conditions would not occur. Thus, the proposed TS change is not expected to lead to significant temporary risk increases. Following the discovery of an unintentionally missed surveillance, SR 4.0.3 requires the licensee to assess temporary risk increases, provided the surveillance cannot be performed within 24 hours from the time it has been discovered. Per SR 4.0.3 Bases, this assessment will be either

qualitative or quantitative depending on the importance of the component affected by the missed surveillance.

3.5 Risk-Informed Configuration Risk Management

RG 1.177 addresses the need for identifying risk-significant configurations resulting from maintenance or other operational activities and taking appropriate compensatory measures to avoid such configurations. The objective of such guidance for this review is to ensure that plant safety will be maintained and monitored during the period of an extended surveillance testing interval (associated with an unintentionally missed surveillance). The licensee proposes to use the program in place to implement the Maintenance Rule to identify “high-risk” configurations resulting from missed surveillance tests in conjunction with outages associated with maintenance activities. It is worth noting that the guidance provided in RG 1.177, with regard to the Configuration Risk Management Program, was used as the basis for developing the guidance contained in RG 1.182 for the 10 CFR 50.65(a)(4) provisions of the Maintenance Rule. This provides additional assurance that the proposed process for evaluating the risk impact of missed surveillances is consistent with guidance provided in RG 1.177, and is, therefore, acceptable.

3.6 Quality of PRA

Once a missed surveillance is discovered and the licensee determines that the surveillance cannot be performed within 24 hours, the licensee will have to use a risk assessment to determine the most prudent course of action. The risk assessment can be done qualitatively or quantitatively, depending on the importance of the component affected by the missed surveillance (missed surveillances for risk important components should be analyzed quantitatively). Such a risk assessment will be consistent with the program to implement the Maintenance Rule guidance to assess and account for both aggregate and temporary risk increases associated with “emergent” plant conditions as well as before undertaking online maintenance or other operational activities.

All licensees must have the capability to assess and manage increases in risk from maintenance activities as required by the Maintenance Rule. Risk assessments performed pursuant to 10 CFR 50.65(a)(4) may use qualitative, quantitative or blended methods. The degree of depth and rigor of the evaluation should be commensurate with the complexity of the proposed configuration to be assessed. Section 11 of NUMARC 93-01 provides guidance for using qualitative, quantitative or blended methods to assess risk. Current inspection programs allow the NRC staff to oversee licensee implementation of 10 CFR 50.65(a)(4) requirements, including the adequacy of pre-maintenance risk assessments performed by licensees.

For the reasons listed below, the NRC staff finds that the same “quality” of PRA or PRA insights used to perform risk assessments pursuant to 10 CFR 50.65(a)(4) is also appropriate when assessing the impact of missed surveillances.

- The number of “emergent” conditions resulting from missed surveillances is very small (in both absolute terms and in comparison to the frequency of “emergent” conditions resulting from equipment failures). The licensee is expected to implement the proposed change to SR 4.0.3 in a manner that ensures that this statement remains valid.

- A missed surveillance is equivalent to a one-time surveillance frequency extension. Therefore, the risk exposure is limited to the duration of the surveillance frequency extension. Risk increases are small compared to similar increases associated with equipment failures. The average (conditional) risk increase, given a missed surveillance, may be comparable to the risk increase from equipment failures. However, due to the rarity of missed surveillances, the average (yearly) risk increase from missed surveillances is expected to be small compared to the risk increase from equipment failures.
- PRA insights indicate that the risk impact from missed surveillances is significant only for a relatively small set of standby equipment. This equipment, such as auxiliary feedwater, high pressure injection pumps, and emergency diesel generators, is located outside containment and generally can be easily tested in a short time, if necessary.
- NRC inspection programs allow NRC staff to oversee the implementation of 10 CFR 50.65(a)(4) requirements, including the adequacy of pre-maintenance risk assessments performed by licensees.

3.7 Summary

The NRC staff review finds that the process proposed by the licensee for addressing missed SRs meets Commission guidance for allowing TS changes. Key elements of the proposed changes are listed below:

- A risk evaluation shall be performed for any surveillance delayed longer than 24 hours, and the risk impact shall be managed.
- The missed surveillance test should be performed at “the first reasonable opportunity.”
- The “first reasonable opportunity” will be determined by taking into consideration the risk impact from delaying the surveillance test as well as the impact on any analysis assumptions, in addition to unit conditions, planning, availability of personnel, and the time required to perform the surveillance.
- A missed surveillance will be treated as an “emergent” condition in the same fashion as other unplanned maintenance activities. The risk impact of the condition will be managed through the program in place to implement 10 CFR 50.65(a)(4) and its implementation guidance (RG 1.182). Rescheduling of missed surveillances pursuant to RG 1.182 will ensure the necessary provisions for managing the risk impact of performing the surveillance in conjunction with other ongoing plant configuration changes.
- The NRC’s operating ROP will provide the framework for inspectors and other NRC staff to review missed surveillances and assess the licensee’s actions and performance. Inspection procedures are in place which will allow NRC staff to oversee the implementation of Maintenance Rule requirements, including the adequacy of pre-maintenance risk assessments performed by licensees.

- A missed surveillance will be placed in the licensee's CAP, thus providing the NRC staff with a means to verify that the number of missed surveillances continues to be very low.
- The number of missed surveillance tests is a very small fraction of the total number of such tests performed at a nuclear plant each year. The proposed change is not intended to be used as an operational convenience to extend surveillance frequencies.
- This process is similar to other improvements that have been made to the TSs that allow the use of a controlled decision-making process by licensees when the process has some high-level regulatory oversight. Two examples of this are the adoption of the Core Operating Limits Report and the Pressure/Temperature Limits Report. In each of these cases, the NRC staff approved the methodology behind the calculation of certain TS parameter limits and then allowed the specific limits to be removed from TSs and controlled by the licensee using the approved methodology. Similarly, for this proposed change, the NRC staff has already approved guidance that outlines a process for complying with 10 CFR 50.65(a)(4) and, therefore, can allow the licensee to use that guidance to determine the most prudent course of action in the case of a missed surveillance.

For these reasons, the NRC staff finds that the proposed TS change, to be implemented in accordance with the above listed key elements, is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Massachusetts and New Hampshire officials were notified of the proposed issuance of the amendment. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, changes SRs, and modifies administrative controls in the TSs. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (68 FR 2804). Accordingly, the amendment changes to SRs 4.0.1 and 4.0.3 (and associated bases) meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). The incorporation of the Bases Control Program meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10)(ii). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: CLIIP SE for Missed Surveillances (66 FR 32400)

Date: May 15, 2003