



Entergy Nuclear Northeast  
Entergy Nuclear Operations, Inc.  
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July 10, 2002  
JAFP-02-0138

T.A. Sullivan  
Vice President, Operations-JAF

U. S. Nuclear Regulatory Commission  
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Rockville, MD 20852

Subject: James A. FitzPatrick Nuclear Power Plant  
Docket No. 50-333  
**Updated Markup and Retyped Technical Specification Pages  
For Proposed Change to the Technical Specifications  
Regarding Allowable MSIV Leakage**

References: 1. Entergy letter, T. Sullivan to USNRC (JAFP-01-0246 Rev. 1) dated  
January 9, 2002 regarding "Proposed Change to the Technical  
Specifications Regarding Allowable MSIV Leakage"

2. USNRC letter, dated July 3, 2002, to M. Kansler (Entergy Nuclear  
Operations, Inc.) transmitting Facility Operating License  
Amendment 274 regarding Improved Technical Specifications

Dear Sir:


This letter and the associated attachments provide replacement pages for those retyped Technical Specification pages, Technical Specification markup pages, and Bases markup pages contained in the Reference 1 proposed license amendment. The replacement pages reflect issuance of Facility Operating License Amendment 274 (Reference 2) subsequent to the Reference 1 submittal.

Attachments 1 and 2 contain the replacement retyped Technical Specification pages and replacement Technical Specification markup pages, respectively.

Attachment 3 contains a draft markup of Technical Specification Bases pages. This is being provided for information only, since the proposed Bases change will be implemented through the Technical Specification Bases Control Program.

There are no new commitments made by this letter. If you have any questions, please contact Mr. R. Plasse at (315) 349-6793.

Very Truly Yours,

  
T. A. Sullivan for  
Vice President, Operations

TAS/EWD/dmr

Attachments as stated

4001

cc:

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U. S. Nuclear Regulatory Commission  
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**Attachment 1 to JAFP-02-0138**

**REVISED TECHNICAL SPECIFICATION PAGES**

Proposed Change to the Technical Specifications Regarding Allowable  
MSIV Leakage

**Entergy Nuclear Operations, Inc.**  
**JAMES A. FITZPATRICK NUCLEAR POWER PLANT**  
Docket No. 50-333  
DPR-59

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.1.3.7	Verify each automatic PCIV actuates to the isolation position on an actual or simulated isolation signal.	24 months
SR 3.6.1.3.8	Verify each reactor instrumentation line EFCV actuates to the isolation position on a simulated instrument line break.	In accordance with the Inservice Testing Program
SR 3.6.1.3.9	Remove and test the explosive squib from each shear isolation valve of the TIP System.	24 months on a STAGGERED TEST BASIS
SR 3.6.1.3.10	Verify combined main steam line leakage rate is $\leq 46$ scfh when tested at $\geq 25$ psig.	In accordance with the Primary Containment Leakage Rate Testing Program
SR 3.6.1.3.11	Verify the leakage rate of each air operated testable check valve associated with the LPCI and CS Systems vessel injection penetrations is within limits.	In accordance with the Primary Containment Leakage Rate Testing Program

**Attachment 2 to JAFP-02-0138**

**MARKED-UP TECHNICAL SPECIFICATION PAGES**

Proposed Change to the Technical Specifications Regarding Allowable  
MSIV Leakage

**Entergy Nuclear Operations, Inc.**  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT  
Docket No. 50-333  
DPR-59

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.6.1.3.7 Verify each automatic PCIV actuates to the isolation position on an actual or simulated isolation signal.	24 months
SR 3.6.1.3.8 Verify each reactor instrumentation line EFCV actuates to the isolation position on a simulated instrument line break.	In accordance with the Inservice Testing Program
SR 3.6.1.3.9 Remove and test the explosive squib from each shear isolation valve of the TIP System.	24 months on a STAGGERED TEST BASIS
SR 3.6.1.3.10 Verify <sup>combined main steam line</sup> leakage rate <del>through each MSIV</del> is <sub>46</sub> $\leq 11.5$ scfh when tested at $\geq 25$ psig.	In accordance with the Primary Containment Leakage Rate Testing Program
SR 3.6.1.3.11 Verify the leakage rate of each air operated testable check valve associated with the LPCI and CS Systems vessel injection penetrations is within limits.	In accordance with the Primary Containment Leakage Rate Testing Program

**Attachment 3 to JAFP-02-0138**

**MARKED-UP TECHNICAL SPECIFICATION BASES PAGES**

Draft Technical Specifications Bases Change Regarding Allowable MSIV  
Leakage

**Entergy Nuclear Operations, Inc.**  
**JAMES A. FITZPATRICK NUCLEAR POWER PLANT**  
Docket No. 50-333  
DPR-59

BASES

SURVEILLANCE  
REQUIREMENTS  
(continued)

SR 3.6.1.3.9

The TIP shear isolation valves are actuated by explosive charges. An in-place functional test is not possible with this design. The explosive squib is removed and tested to provide assurance that the valves will actuate when required. The replacement charge for the explosive squib shall be from the same manufactured batch as the one fired or from another batch that has been certified by having one of the batch successfully fired. The Frequency of 24 months on a STAGGERED TEST BASIS is considered adequate given the administrative controls on replacement charges and the frequent checks of circuit continuity (SR 3.6.1.3.4).

SR 3.6.1.3.10

46

The combined  
main steam line  
leakage rate

The analyses in Reference 11 are based on leakage that is more than the specified leakage rate. ~~Leakage through each MSIV must be  $\leq 11.5$  scfh when tested at  $\geq 25$  psig.~~ This ensures that MSIV leakage is properly accounted for in determining the overall primary containment leakage rate. The Frequency is in accordance with the Primary Containment Leakage Rate Testing Program.

SR 3.6.1.3.11

Surveillance of each air operated testable check valve associated with the LPCI and CS Systems vessel injection penetrations provides assurance that the resulting radiation dose rate that would result if the reactor coolant were released to the reactor building at the specified limit will be small (Ref. 12). The acceptance criteria for each air operated testable check valve associated with the LPCI and CS Systems vessel injection penetrations is  $< 10$  gpm when hydrostatically tested at  $\geq 1035$  psig or  $< 10$  scfm when pneumatically tested at  $\geq 45$  psig, at ambient temperature (Ref. 12). The leakage rates must be demonstrated in accordance with the leakage rate test Frequency required by the Primary Containment Leakage Rate Testing Program.

REFERENCES

1. UFSAR, Section 14.6.
2. UFSAR, Section 14.5.2.3.
3. UFSAR, Section 6.5.3.2.
4. UFSAR, Section 14.8.2.1.2.

(continued)