September 13, 1995

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
San Onofre Nuclear Generating Station
P. O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M90059) AND UNIT NO. 3 (TAC NO. M90060)

Dear Mr. Ray:

The Commission has issued the enclosed Amendment No. 125 to Facility Operating License No. NPF-10 and Amendment No. 114 to Facility Operating License No. NPF-15 for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated May 20, 1994.

These amendments revise the TS and corresponding Bases to support the addition of the component cooling water surge tank backup nitrogen supply system.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original Signed By
Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures: 1. Amendment No. 125 to NPF-10
2. Amendment No. 114 to NPF-15
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION

Docket File	LHurley, RIV
PUBLIC	KPerkins, WCFO
PDIV-2 Reading	EPeyton
JRoe	GHill (4), T5C3
EGA1	ACRS (4), T2E26
WBateman	JBianchi, WCFO (2)
MFields	OGC, O15B18
CGrimes, O11E22	CMcCracken

DOCUMENT NAME: S090059.AMD

OFC	PDIV-2/LA	PDIV-2/PM	BC:SPLB	OGC
NAME	EPeyton	MFields:ye	CMcCracken	RBachmann
DATE	8/28/95	8/29/95	8/3/95	9/1/95

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September 13, 1995

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 13, 1995

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
San Onofre Nuclear Generating Station
P. O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M90059) AND UNIT NO. 3 (TAC NO. M90060)

Dear Mr. Ray:

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These amendments revise the TS and corresponding Bases to support the addition of the component cooling water surge tank backup nitrogen supply system.

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Mel B. Fields

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Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures: 1. Amendment No. 125 to NPF-10
2. Amendment No. 114 to NPF-15
3. Safety Evaluation

cc w/encls: See next page

cc w/encls:
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San Clemente, California 92674-0128

Chairman, Board of Supervisors
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Mayor
City of San Clemente
100 Avenida Presidio
San Clemente, California 92672

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee) dated May 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 125, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of the date of issuance to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 13, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 7-11	3/4 7-11
B 3/4 7-3b	B 3/4 7-3b

3/4.7.3 COMPONENT COOLING WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.3 At least two independent component cooling water loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With only one component cooling water loop OPERABLE, restore at least two loops to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With either one or both trains of the Backup Nitrogen Supply (BNS) system inoperable, within 8 hours restore the BNS system train(s) to OPERABLE status or declare the associated CCW loop(s) inoperable.

SURVEILLANCE REQUIREMENTS

4.7.3.1 At least two component cooling water loops shall be demonstrated OPERABLE:

a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.

b. At least once per refueling interval during shutdown, by verifying that each automatic valve servicing safety-related equipment actuates to its correct position and each component cooling water pump starts automatically on an SIAS test signal.

4.7.3.2 The BNS system shall be demonstrated OPERABLE:

a. At least once per 7 days by verifying that at least nine nitrogen gas bottles are installed with a minimum average bottle pressure of 4232 psig.

b. At least once per refueling interval by verifying that the third stage pressure regulator of the BNS system is set at 55 psig (± 1.5 psi).

3/4.7.4 SALT WATER COOLING SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.4 At least two independent salt water cooling loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With only one salt water cooling loop OPERABLE, restore at least two loops to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.4 At least two salt water cooling loops shall be demonstrated OPERABLE:

a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed or otherwise secured in position, is in its correct position.

b. At least once per refueling interval during shutdown, by verifying that each automatic valve servicing safety-related equipment actuates to its correct position and each salt water cooling pump starts automatically on an SIAS test signal.

3/4.7.1.6 ATMOSPHERIC DUMP VALVES (Continued)

The provisions of Specification 3.0.4 in MODES 2, 3, and 4 do not apply when only one ADV is inoperable, and the ADV can be made OPERABLE within the allowed action times. However, with two inoperable ADVs the plant must be placed on shutdown cooling. Therefore, the provisions of Specification 3.0.4 do apply with two inoperable ADVs.

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

The limitation on secondary side steam generator pressure and temperature ensures that the pressure induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The limitations of 70°F and 200 psig are based on a steam generator RT_{NDT} of 40°F and are sufficient to prevent brittle fracture.

3/4.7.3 COMPONENT COOLING WATER SYSTEM

The OPERABILITY of the Component Cooling Water (CCW) system ensures that sufficient cooling capacity is available for continued operation of safety related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the accident analyses.

The CCW system is normally pressurized to maintain the CCW system water-solid using nitrogen gas supplied to the CCW surge tank by the non-safety related Auxiliary Gas system. Makeup water to the surge tank is normally provided by the non-safety related, Nuclear Service Water system to compensate for normal system leakage.

Following a Design Basis Event, both the non-safety related Auxiliary Gas System and Nuclear Service Water system are assumed to be unavailable. A postulated Design Basis Event could result in CCW system voiding and a subsequent water hammer. The Backup Nitrogen Supply (BNS) system is an independent, safety related, Seismic Category I source of pressurized nitrogen for both CCW surge tanks. The BNS system is designed to minimize CCW system high-point voiding by maintaining the CCW critical loops water-solid during Design Basis Event mitigation.

BNS system OPERABILITY ensures that both CCW surge tanks will be pressurized for at least seven days following a Design Basis Event without bottle changeout. The BNS system is required to be OPERABLE whenever the associated train of CCW is required to be OPERABLE. The BNS system surveillance requirements provide adequate assurance that BNS system OPERABILITY will be maintained.

3/4.7.4 SALT WATER COOLING SYSTEM

The OPERABILITY of the salt water cooling system ensures that sufficient cooling capacity is available for continued operation of equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the accident analyses.

3/4.7.5 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM

The OPERABILITY of the control room emergency air cleanup system ensures that 1) the ambient air temperature does not exceed the allowable temperature for continuous duty rating for the equipment and instrumentation cooled by this system and 2) the control room will remain habitable for operations personnel during and following all credible accident conditions. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criteria 19 of Appendix A, 10 CFR 50.

Cumulative operation of the system with the heaters on for at least 10 hours over a 31 day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee) dated May 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 114, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of the date of issuance to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: September 13, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 7-12	3/4 7-12
B 3/4 7-3b	B 3/4 7-3b

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

LIMITING CONDITION FOR OPERATION

3.7.2 The temperature of the secondary coolant in the steam generators shall be greater than 90°F when the pressure of the secondary coolant in the steam generator is greater than 200 psig.

APPLICABILITY: At all times.

ACTION:

With the requirements of the above specification not satisfied:

a. Reduce the steam generator pressure of the secondary side to less than or equal to 200 psig within 30 minutes, and

b. Perform an engineering evaluation to determine the effect of the overpressurization on the structural integrity of the steam generator. Determine that the steam generator remains acceptable for continued operation prior to increasing its temperatures above 200°F.

SURVEILLANCE REQUIREMENTS

4.7.2 The pressure in the secondary side of the steam generators shall be determined to be less than 200 psig at least once per hour when the temperature of the secondary coolant is less than 90°F.

3/4.7.3 COMPONENT COOLING WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.3 At least two independent component cooling water loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With only one component cooling water loop OPERABLE, restore at least two loops to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With either one or both trains of the Backup Nitrogen Supply (BNS) system inoperable, within 8 hours restore the BNS system train(s) to OPERABLE status or declare the associated CCW loop(s) inoperable.

SURVEILLANCE REQUIREMENTS

4.7.3.1 At least two component cooling water loops shall be demonstrated OPERABLE:

a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.

b. At least once per refueling interval during shutdown, by verifying that each automatic valve servicing safety-related equipment actuates to its correct position and each component cooling water pump starts automatically on an SIAS test signal.

4.7.3.2 The BNS system shall be demonstrated OPERABLE:

a. At least once per 7 days by verifying that at least nine nitrogen gas bottles are installed with a minimum average bottle pressure of 4232 psig.

b. At least once per refueling interval by verifying that the third stage pressure regulator of the BNS system is set at 55 psig (± 1.5 psi).

3/4.7.1.6 ATMOSPHERIC DUMP VALVES (Continued)

the ADVs are subject to inservice testing per Surveillance 4.7.1.6.3, the frequency of Surveillance 4.7.1.6.1 is based on the length of a fuel cycle.

The provisions of Specification 3.0.4 in MODES 2, 3, and 4 do not apply when only one ADV is inoperable, and the ADV can be made OPERABLE within the allowed action times. However, with two inoperable ADVs the plant must be placed on shutdown cooling. Therefore, the provisions of Specification 3.0.4 do apply with two inoperable ADVs.

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

The limitation on secondary side steam generator pressure and temperature ensures that the pressure-induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The limitations of 90°F and 200 psig are based on a steam generator RT_{NDT} of 60°F and are sufficient to prevent brittle fracture.

3/4.7.3 COMPONENT COOLING WATER SYSTEM

The OPERABILITY of the Component Cooling Water (CCW) system ensures that sufficient cooling capacity is available for continued operation of safety related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the accident analyses.

The CCW system is normally pressurized to maintain the CCW system water-solid using nitrogen gas supplied to the CCW surge tank by the non-safety related Auxiliary Gas System. Makeup water to the surge tank is normally provided by the non-safety related, Nuclear Service Water system to compensate for normal system leakage.

Following a Design Basis Event, both the non-safety related Auxiliary Gas system and Nuclear Service Water system are assumed to be unavailable. A postulated Design Basis Event could result in CCW system voiding and a subsequent water hammer. The Backup Nitrogen Supply (BNS) system is an independent, safety related, Seismic Category I source of pressurized nitrogen for both CCW surge tanks. The BNS system is designed to minimize CCW system high-point voiding by maintaining the CCW critical loops water-solid during Design Basis Event mitigation.

BNS system OPERABILITY ensures that both CCW surge tanks will be pressurized for at least seven days following a Design Basis Event without bottle changeout. The BNS system is required to be OPERABLE whenever the associated train of CCW is required to be OPERABLE. The BNS system surveillance requirements provide adequate assurance that BNS system OPERABILITY will be maintained.

3/4.7.4 SALT WATER COOLING SYSTEM

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3/4.7.5 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM

The OPERABILITY of the control room emergency air cleanup system ensures that 1) the ambient air temperature does not exceed the allowable temperature for continuous duty rating for the equipment and instrumentation cooled by this system and 2) the control room will remain habitable for operations personnel during and following all credible accident conditions. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of Appendix A, 10 CFR 50.

Cumulative operation of the system with the heaters on for at least 10 hours over a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-10

AND AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

By letter dated May 20, 1994, Southern California Edison Company, et al. (SCE or the licensee) submitted a request for changes to the Technical Specifications (TS) for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3. The proposed changes would revise TS 3/4.7.3, "Component Cooling Water System," and the corresponding Bases to support the addition of the component cooling water (CCW) surge tank backup nitrogen supply (BNS) system.

2.0 BACKGROUND

The licensee, in an assessment following the NRC's Safety System Functional Inspection of the CCW and salt water cooling systems, determined that the CCW system was susceptible to water hammer during flow transients after certain design basis events. In a letter to the NRC dated January 4, 1990, the licensee indicated its intent to implement three modifications to remedy the water hammer susceptibility, including the addition of a qualified BNS system which would be connected to the surge tanks of the CCW system.

3.0 EVALUATION

At San Onofre, the CCW system includes two independent critical cooling loops and one non-critical cooling loop (NCL). Each critical cooling loop is 100 percent capacity and contains a surge tank, pump, and heat exchanger. The NCL is isolated upon a containment isolation actuation signal or a low-low surge tank level signal. During normal plant operation, the nuclear service water system supplies makeup water to the surge tanks, and the auxiliary gas system supplies nitrogen gas to the surge tanks to ensure that the CCW system is maintained water solid during normal operation. Neither of these systems are safety related nor do they support emergency operations.

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The BNS system would maintain CCW system pressure for cases where a break occurs in the NCL loop and the normal, nonsafety-related nitrogen supply cannot be credited. The addition of the BNS system provides the SONGS Units 2 and 3 CCW systems with a seismic Category I source of nitrogen with an independent BNS train for each CCW critical loop. Each train of the BNS system contains a bank of ten 6000 psig nitrogen bottles installed in dedicated bottle racks and connected by a common discharge header. Bottle replacement is facilitated by provision of a pressure gauge, an isolation valve, and a check valve for each bottle.

The BNS system is designed to maintain the surge tank pressure for a minimum of seven days following a design basis event without operator action. A seven-day period of BNS system operation without action is based on reducing the post-LOCA dose rate in the BNS bottle changeout area to below 10 CFR 50, Appendix A, General Design Criterion 19 limits. The seven days of BNS system operation will require approximately 2974 standard cubic feet (scf) of nitrogen which corresponds to a minimum initial average bottle pressure of 4132 pounds per square inch gauge (psig). The nitrogen inventory is administratively controlled by the proposed TS bottle pressure surveillance. Proposed TS 4.7.3.2a requires the average bottle pressure of nine of the ten bottles to be at least 4232 psig, which is 100 psi greater than the operability requirement of 4132 psig. This 100 psi margin accounts for seven days of maximum expected BNS system leakage.

The BNS system is designed with four stages of pressure regulation such that the normal nitrogen supply can be used without actuating the BNS system and to offer protection against over-pressurization of downstream piping and the surge tank. The auxiliary gas system nitrogen supply pressure is normally available at 85 to 105 psig. At the supply pressure of 85 psig and a maximum expected nitrogen flow rate during normal operation of 230 scfm the pressure downstream of the third stage regulator will be approximately 59.5 psig. The third stage pressure regulator setpoint of 55 psig (± 1.5 psi) will assure that the BNS system will remain isolated while the auxiliary gas system is functional, while being capable of maintaining the surge tank pressure above 27.4 psig should the normal nitrogen supply become unavailable. The BNS system third stage pressure regulator setpoint will be controlled by the proposed TS 4.7.3.2b.

The NRC staff has reviewed the design features of the BNS system, including the proposed TS controlling the operation of the system, and concludes that the BNS system will perform its intended safety function. The staff therefore finds acceptable the proposed addition of the BNS system to the San Onofre Units 2 and 3 plant design and TS.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 45034). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: M. A. Miller
J. Kennedy

Date: September 13, 1995