August 1, 1996

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
P.O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M95786) AND UNIT NO. 3 (TAC NO. M95787)

Dear Mr. Ray:

The Commission has issued the enclosed Amendment No. 130 to Facility Operating License No. NPF-10 and Amendment No. 119 to Facility Operating License No. NPF-15 for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated June 3, 1996, as superseded by letter dated June 25, 1996.

These amendments revise Improved Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," and Improved TS 5.5.2.13, "Diesel Fuel Oil Testing Program." Specifically, the number of instruments required to measure reactor coolant inlet temperature (T_{Cold}), and reactor coolant outlet temperature (T_{Hot}), will be revised from two per loop to two (with one cold leg indication and one hot leg indication per steam generator). These changes to the Improved TS reinstate provisions of the current SONGS Units 2 and 3 TS revised as part of NRC Amendment Nos. 127 and 116 for SONGS Units 2 and 3 (referred to herein as the Improved TS).

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures: 1. Amendment No. 130 to NPF-10
2. Amendment No. 119 to NPF-15
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION

Docket File	EPeyton
PUBLIC	JRoe
PDIV-2 Reading	GHill (4), T5C3
EGA1	ACRS, T2E26
WBateman	JBianchi, WCFO (2)
MFields	OGC, O15B18
CGrimes, O11E22	CMH2 (SE)
LHurley, RIV	KPerkins, WCFO
JKilcrease, WCFO	JWermiel

DOCUMENT NAME: SO95786.AMD

OFC	LA:PDIV-2	PDIV-2	HICB	OGC	PDIV-2
NAME	EPeyton	MFields	JWermiel	RBachmann	WBateman
DATE	7/3/96	7/3/96	7/3/96	7/8/96	7/30/96

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August 1, 1996

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Executive Vice President
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These amendments revise Improved Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," and Improved TS 5.5.2.13, "Diesel Fuel Oil Testing Program." Specifically, the number of instruments required to measure reactor coolant inlet temperature (T_{Cold}), and reactor coolant outlet temperature (T_{Hot}), will be revised from two per loop to two (with one cold leg indication and one hot leg indication per steam generator). These changes to the Improved TS reinstate provisions of the current SONGS Units 2 and 3 TS revised as part of NRC Amendment Nos. 127 and 116 for SONGS Units 2 and 3 (referred to herein as the Improved TS).

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LHurley, RIV	KPerkins, WCFO
JKilcrease, WCFO	JWermiel

DOCUMENT NAME: S095786.AMD

OFC	LA:PDIV-2	PDIV-2	HICB	OGC	PDIV-2
NAME	EPeyton	MFields	JWermiel	RBachmann	WBateman
DATE	7/3/96	7/ /96	7/3 /96	7/8 /96	7/30/96

OFFICIAL RECORD COPY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 1, 1996

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
P.O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M95786) AND UNIT NO. 3 (TAC NO. M95787)

Dear Mr. Ray:

The Commission has issued the enclosed Amendment No. 130 to Facility Operating License No. NPF-10 and Amendment No. 119 to Facility Operating License No. NPF-15 for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated June 3, 1996, as superseded by letter dated June 25, 1996.

These amendments revise Improved Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," and Improved TS 5.5.2.13, "Diesel Fuel Oil Testing Program." Specifically, the number of instruments required to measure reactor coolant inlet temperature (T_{Cold}), and reactor coolant outlet temperature (T_{Hot}), will be revised from two per loop to two (with one cold leg indication and one hot leg indication per steam generator). These changes to the Improved TS reinstate provisions of the current SONGS Units 2 and 3 TS revised as part of NRC Amendment Nos. 127 and 116 for SONGS Units 2 and 3 (referred to herein as the Improved TS).

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures: 1. Amendment No. 130 to NPF-10
2. Amendment No. 119 to NPF-15
3. Safety Evaluation

cc w/encls: See next page

cc w/encls:
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San Onofre Nuclear Generating Station
P. O. Box 128
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Chairman, Board of Supervisors
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c/o U.S. Nuclear Regulatory Commission
Post Office Box 4329
San Clemente, California 92674

Mayor
City of San Clemente
100 Avenida Presidio
San Clemente, California 92672

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee) dated June 3, 1996, as superseded by application dated June 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 130 , are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented by August 9, 1996.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 1, 1996

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Revise the issued but not yet implemented Appendix A Improved Technical Specifications (ITS) by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

REMOVE	INSERT
ITS 3.3-47	ITS 3.3-47
ITS 5.0-20	ITS 5.0-20
ITS B 3.3-162	ITS B 3.3-162

Table 3.3.11-1 (page 1 of 1)
Post Accident Monitoring Instrumentation

	FUNCTION	REQUIRED CHANNELS	CONDITIONS REFERENCED FROM REQUIRED ACTION F.1
1.	Excore Neutron Flux	2	G
2.	Reactor Coolant System Hot Leg Temperature	2 (1 per steam generator)	G
3.	Reactor Coolant System Cold Leg Temperature	2 (1 per steam generator)	G
4.	Reactor Coolant System Pressure (wide range)	2	G
5.	Reactor Vessel Water Level	2[d]	H
6.	Containment Sump Water Level (wide range)	2	G
7.	Containment Pressure (wide range)	2	G
8.	Containment Isolation Valve Position	2 per penetration flow path[a][b]	G
9.	Containment Area Radiation (high range)	2	H
10.	Containment Hydrogen Monitors	2	G
11.	Pressurizer Level	2	G
12.	Steam Generator Water Level (wide range)	2 per steam generator	G
13.	Condensate Storage Tank Level	2	G
14.	Core Exit Temperature – Quadrant 1	2[c]	G
15.	Core Exit Temperature – Quadrant 2	2[c]	G
16.	Core Exit Temperature – Quadrant 3	2[c]	G
17.	Core Exit Temperature – Quadrant 4	2[c]	G
18.	Auxiliary Feedwater Flow	1 per steam generator	G
19.	Containment Pressure (narrow range)	2	G
20.	Reactor Coolant System Subcooling Margin Monitor	2	G
21.	Pressurizer Safety Valve Position Indication	1 per valve	G
22.	Containment Temperature	2	G
23.	Containment Water Level (narrow range)	2	G
24.	HPSI Flow Cold Leg	1 per cold leg	G
25.	HPSI Flow Hot Leg	1 per hot leg	G
26.	Steam Line Pressure	2 per steam generator	G
27.	Refueling Water Storage Tank Level	2	G

(a) Not required for isolation valves whose associated penetration is isolated by at least one closed and de-activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(c) A channel consists of two or more core exit thermocouples.

(d) A channel consists of eight sensors in a probe. A channel is OPERABLE if four or more sensors, one sensor in the upper head and three sensors in the lower head are OPERABLE.

5.5 Procedures, Programs, and Manuals

5.5.2.12 Ventilation Filter Testing Program (VFTP) (continued)

The provisions of Technical Specification Surveillance Requirement 3.0.2 and Technical Specification Surveillance Requirement 3.0.3 are applicable to the VFTP test frequencies.

5.5.2.13 Diesel Fuel Oil Testing Program

This program implements required testing of both new fuel oil and stored fuel oil. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM standards. The purpose of the program is to establish the following:

a. At least once per 92 days and from new fuel oil prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to 0.05 volume percent, an API gravity or an absolute specific gravity within limits, and a kinematic viscosity @ 40 C of greater than or equal to 1.9 but less than or equal to 4.1 when tested in accordance with ASTM-D975-81.

b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D4057-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.

BASES

LCO (continued)

2, 3. Reactor Coolant System (RCS) Hot and Cold Leg Temperature

RCS Hot and Cold Leg Temperatures are Category I variables provided for verification of core cooling and long term surveillance.

Reactor outlet temperature inputs to the PAMI are provided by two fast response resistance elements and associated transmitters.

4. Reactor Coolant System Pressure (wide range)

RCS Pressure (wide range) is a Category I variable, provided for verification of core cooling and RCS integrity long term surveillance.

The pressure transmitters are located inside the containment. Redundant monitoring capability is provided by two trains of instrumentation.

Operator actions to maintain a controlled cooldown, such as adjusting steam generator pressure or level, would use this indication. Furthermore, RCS pressure is one factor that may be used in decisions to terminate reactor coolant pump operation.

5. Reactor Vessel Water Level

Reactor Vessel Water Level is provided for verification and long term surveillance of core cooling.

The Reactor Vessel Water Level Monitoring System provides a direct measurement of the collapsed liquid level above the fuel alignment plate. The collapsed level represents the amount of liquid mass that is in the reactor vessel above the core.

(continued)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee) dated June 3, 1996, as superseded by application dated June 25, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 119 , are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented by August 9, 1996.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 1, 1996

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Revise the issued but not yet implemented Appendix A Improved Technical Specifications (ITS) by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

REMOVE	INSERT
ITS 3.3-47	ITS 3.3-47
ITS 5.0-20	ITS 5.0-20
ITS B 3.3-162	ITS B 3.3-162

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4.	Reactor Coolant System Pressure (wide range)	2	G
5.	Reactor Vessel Water Level	2(d)	H
6.	Containment Sump Water Level (wide range)	2	G
7.	Containment Pressure (wide range)	2	G
8.	Containment Isolation Valve Position	2 per penetration flow path(a)(b)	G
9.	Containment Area Radiation (high range)	2	H
10.	Containment Hydrogen Monitors	2	G
11.	Pressurizer Level	2	G
12.	Steam Generator Water Level (wide range)	2 per steam generator	G
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18.	Auxiliary Feedwater Flow	1 per steam generator	G
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20.	Reactor Coolant System Subcooling Margin Monitor	2	G
21.	Pressurizer Safety Valve Position Indication	1 per valve	G
22.	Containment Temperature	2	G
23.	Containment Water Level (narrow range)	2	G
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This program implements required testing of both new fuel oil and stored fuel oil. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM standards. The purpose of the program is to establish the following:

a. At least once per 92 days and from new fuel oil prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to 0.05 volume percent, an API gravity or an absolute specific gravity within limits, and a kinematic viscosity @ 40 C of greater than or equal to 1.9 but less than or equal to 4.1 when tested in accordance with ASTM-D975-81.

b. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D4057-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM-D2276-83, Method A.

BASES

LCO (continued)

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(continued)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NO. NPF-10

AND AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

By application dated June 3, 1996, as superseded by application dated June 25, 1996, Southern California Edison Company (SCE or the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License Nos. NPF-10 and NPF-15) for San Onofre Nuclear Generating Station (SONGS), Unit Nos. 2 and 3. The proposed changes would revise Improved Technical Specification (TS) 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," and Improved TS 5.5.2.13, "Diesel Fuel Oil Testing Program." These TS changes reinstate provisions of the current SONGS Units 2 and 3 TS revised as part of NRC Amendment Nos. 127 and 116 for SONGS Units 2 and 3 (referred to herein as the Improved TS).

2.0 BACKGROUND

NRC Amendment Nos. 127 and 116, dated February 9, 1996, approved a license amendment request by SCE that adopted the recommendations of NUREG-1432, "Standard Technical Specifications - Combustion Engineering Plants." These amendments revised, in their entirety, the TS and the Bases for SONGS Units 2 and 3. These Improved TS are to be implemented by the licensee no later than August 9, 1996.

During preparation of the procedure changes necessary to implement NRC Amendment Nos. 127 and 116, the licensee identified certain provisions of the current TS that were not properly incorporated into the Improved TS. In its letter dated June 25, 1996, the licensee requested that these provisions be restored into the Improved TS.

3.0 EVALUATION

The licensee has proposed changes to TS 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," and TS 5.5.2.13, "Diesel Fuel Oil Testing Program," of the Improved TS. The changes would reinstate provisions of the current SONGS Units 2 and 3 TS. Each of the proposed changes to the Improved TS are evaluated in detail below.

Improved TS 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)"

Under the current TS, TS 3/4.3.3.6, "Accident Monitoring Instrumentation," requires operability of two channels of reactor coolant inlet temperature (T_{Cold}) and two channels of Reactor Coolant Outlet Temperature (T_{Hot}). Improved TS 3.3.11, "Post Accident Monitoring Instrumentation (PAMI)," which is based on the model TS contained in NUREG-1432, identifies two channels of reactor coolant inlet temperature (T_{Cold}) per loop, and two channels of reactor coolant outlet temperature (T_{Hot}) per loop. Therefore, there was an unintentional increase in the number of instruments required when the licensee incorporated this section of NUREG-1432 into the SONGS Improved TS. The currently installed instrumentation satisfies the current TS, but cannot satisfy the specification inadvertently directly transcribed from NUREG-1432.

The SONGS requirements for PAMI are based on analyses performed to support Regulatory Guide (RG) 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2, 1980. T_{Cold} and T_{Hot} are identified as plant specific parameters considered Type A variables. SCE identified that the instrumentation to measure these parameters deviates from the RG in that the measurement range is 50°F to 710°F rather than the specified range of 50°F to 750°F. This was found acceptable by the NRC in its May 26, 1987, letter, "Safety Evaluation for Conformance to Regulatory Guide 1.97."

The only difference between the SONGS current TS and the NUREG TS (and current version of the Improved TS) for these parameters is the number of instruments per loop, and RG 1.97 provides guidance on the redundancy needed for these parameters. RG 1.97 states, "Where failure of one accident-monitoring channel results in information ambiguity (that is, the redundant displays disagree) that could lead operators to defeat or fail to accomplish a required safety function, additional information should be provided to allow the operators to deduce the actual conditions in the plant. This may be accomplished by providing additional independent channels of information of the same variable (addition of an identical channel) or by providing an independent channel to monitor a different variable that bears a known relationship to the multiple channels (addition of a diverse channel)."

The redundancy recommendations of RG 1.97 are met for T_{Cold} and T_{Hot} at SONGS through the use of diverse channels. The cold leg temperature can be derived by measuring the steam generator pressure and converting to temperature using the steam tables. The SONGS steam generator pressure instrumentation fully meet the RG 1.97 Category 1 recommendations and are acceptable diverse indications for providing operators with unambiguous information on

temperature conditions in the cold legs. The hot leg temperature can be directly determined from the core exit thermocouples. The core exit thermocouples fully meet the RG 1.97 Category 1 recommendations, and are acceptable diverse indications for providing operators with unambiguous information on temperature conditions in the hot legs.

Therefore, the staff finds acceptable the restoration of the current TS requirement of two cold leg and two hot leg RCS temperature channels. The proposed change to the Improved TS will clarify that the two channels required include one cold leg indication and one hot leg indication per steam generator. The Bases for TS 3.3.11 will also be revised accordingly.

Improved TS 5.5.2.13, "Diesel Fuel Oil Testing Program"

Improved TS 5.5.2.13 is the diesel fuel oil testing program required by Surveillance Requirement (SR) 3.8.3.3 of Improved TS 3.8.3, "Diesel Fuel Oil, Lube Oil, and Starting Air." Section 5.5.2.13.a of the program includes various sampling and testing requirements, to be performed at least once every 92 days, and from new fuel oil prior to addition to the storage tanks. One of the tests verifies the kinematic viscosity. The limit currently specified in 5.5.2.13.a, is "greater than or equal to 4.1," which is incorrect. The Bases for SR 3.8.3 specifies the correct limit of "greater than or equal to 1.9, but less than or equal to 4.1." The range of 1.9 to 4.1 is also the acceptable viscosity range specified in ASTM-D975-81. The proposed change would revise the viscosity limit specified in the 5.5.2.13.a to be consistent with the Bases to SR 3.8.3.3. Also, a typographical error in paragraph b is corrected. The ASTM standard for sampling fuel oil is restored to ASTM-D4057-81.

The proposed changes restore the diesel fuel oil requirements to the existing requirements contained in the current TS. SR 4.8.1.1.2.c.1 of the current TS specifies the correct range of 1.9 to 4.1 for the kinematic viscosity, and SR 4.8.1.1.2.c.2 of the current TS specifies that a sample of fuel oil be obtained in accordance with ASTM-D4057-81. The staff finds acceptable the proposed changes to Improved TS 5.5.2.13.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no

public comment on such finding (61 FR 34452). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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