Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
P.O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M90658) AND UNIT NO. 3 (TAC NO. M90659)

Dear Mr. Ray:

The Commission has issued the enclosed Amendment No. 123 to Facility Operating License No. NPF-10 and Amendment No. 112 to Facility Operating License No. NPF-15 for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated August 26, 1994.

These amendments revise TS 3/4.7.5, "Control Room Emergency Air Cleanup System," to provide an exception to Limiting Condition for Operation (LCO) 3.0.4 for Modes 5 and 6 and for a defueled configuration. These amendments also add the applicability statement "or during movement of irradiated fuel assemblies."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures:
1. Amendment No. 123to NPF-10
2. Amendment No. 112to NPF-15
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION

Docket File	EPeyton
PUBLIC	GHill (4), T5C3
OPA, O2G5	OC/LFDCB, T9E10
AAdensam	PDIV-2 Reading
TQuay	OGC, O15B18
CGrimes, O11E22	ACRS (4), T2E26
Region IV	MFields
RIV, WCFO	KPerkins, RIV/WCFO
DSkay	CMcCracken

DOCUMENT NAME: S090658.AMD

OFC	PDIV-2	PDIV-2	SPLB	OGC RMW
NAME	EPeyton	MFields:pk	CMcCracken	No legal objection RWeisman
DATE	7/6/95	7/6/95	7/7/95	7/18/95

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Executive Vice President
Southern California Edison Company
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San Clemente, California 92674-0128

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Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
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NAME	EPeyton	MFields:pk	CMcCracken	No legal objection RWeisman
DATE	7/6/95	7/6/95	7/7/95	7/18/95

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 26, 1995

Mr. Harold B. Ray
Executive Vice President
Southern California Edison Company
P.O. Box 128
San Clemente, California 92674-0128

SUBJECT: ISSUANCE OF AMENDMENT FOR SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2 (TAC NO. M90658) AND UNIT NO. 3 (TAC NO. M90659)

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These amendments revise TS 3/4.7.5, "Control Room Emergency Air Cleanup System," to provide an exception to Limiting Condition for Operation (LCO) 3.0.4 for Modes 5 and 6 and for a defueled configuration. These amendments also add the applicability statement "or during movement of irradiated fuel assemblies."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

William H. Bateman for

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-361
and 50-362

Enclosures:
1. Amendment No. 123 to NPF-10
2. Amendment No. 112 to NPF-15
3. Safety Evaluation

cc w/encls: See next page

cc w/encls:
Mr. R. W. Krieger, Vice President
Southern California Edison Company
San Onofre Nuclear Generating Station
P. O. Box 128
San Clemente, California 92674-0128

Chairman, Board of Supervisors
County of San Diego
1600 Pacific Highway, Room 335
San Diego, California 92101

Alan R. Watts, Esq.
Rourke & Woodruff
701 S. Parker St. No. 7000
Orange, California 92668-4702

Mr. Sherwin Harris
Resource Project Manager
Public Utilities Department
City of Riverside
3900 Main Street
Riverside, California 92522

Dr. Harvey Collins, Chief
Division of Drinking Water and
and Environmental Management
California Department of Health Services
P. O. Box 942732
Sacramento, California 94234-7320

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Harris Tower & Pavilion
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Mr. Richard Krumvieda
Manager, Nuclear Department
San Diego Gas & Electric Company
P.O. Box 1831
San Diego, California 92111

Mr. Steve Hsu
Radiologic Health Branch
State Department of Health Services
Post Office Box 942732
Sacramento, California 94234

Resident Inspector/San Onofre NPS
c/o U.S. Nuclear Regulatory Commission
Post Office Box 4329
San Clemente, California 92674

Mayor
City of San Clemente
100 Avenida Presidio
San Clemente, California 92672

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee) dated August 26, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 123 , are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 26, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by the Amendment number and contains marginal lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

REMOVE	INSERT
3/4 7-13	3/4 7-13

3/4.7.5 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM*

LIMITING CONDITION FOR OPERATION

3.7.5 Two independent control room emergency air cleanup systems shall be OPERABLE.

APPLICABILITY: ALL MODES or during movement of irradiated fuel assemblies.

ACTION:

Unit 2 or 3 in MODE 1, 2, 3 or 4:

With one control room emergency air cleanup system inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

Units 2 or 3 in MODE 5 or 6; or defueled when moving irradiated fuel assemblies:

a. With one control room emergency air cleanup system inoperable, restore the inoperable system to OPERABLE status within 7 days or initiate and maintain operation of the remaining OPERABLE control room emergency air cleanup system in the recirculation mode.

b. With both control room emergency air cleanup systems inoperable, or with the OPERABLE control room emergency air cleanup system required to be in the recirculation mode by ACTION (a), not capable of being powered by an OPERABLE emergency power source, suspend all operations involving CORE ALTERATIONS or positive reactivity changes; or movement of irradiated fuel assemblies.

c. The provisions of Specification 3.0.3 are not applicable in MODE 6.

d. The provisions of Specification 3.0.4 are not applicable when entering MODES 5, 6, or defueled configuration.

SURVEILLANCE REQUIREMENTS

4.7.5 Each control room emergency air cleanup system shall be demonstrated OPERABLE:

a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 110°F.

b. At least once per 31 days on a STAGGERED TEST BASIS by initating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 10 hours with the heaters on.

c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:

   1. Verifying that with the system operating at a flow rate of 35485 cfm ± 10% for the air conditioning unit, and 2050 ± 150 cfm for the ventilation unit and recirculating through the respective HEPA filters and charcoal adsorbers, leakage through the system diverting valves is less than or equal to 1% air conditioning unit and 1% ventilation unit when the system is tested by admitting cold DOP at the respective intake.

*Shared system with San Onofre - Unit 3.

SURVEILLANCE REQUIREMENTS (Continued)

2. Verifying that the cleanup system satisfies the in-place testing acceptance criteria and uses the test procedures of Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 2050 ± 150 cfm for the ventilation unit and 35,485 cfm ± 10% for the air conditioning unit.

3. Verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

4. Verifying a system flow rate of 2050 ± 150 cfm for the ventilation unit and 35,485 cfm ± 10% for the air conditioning unit during system operation when tested in accordance with ANSI N510-1975.

d. After every 720 hours of charcoal adsorber operation by verifying within 31 days after removal that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978.

e. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 7.0 inches Water Gauge ventilation unit and less than 7.3 inches Water Gauge air conditioning unit while operating the system at a flow rate of 2050 ± 150 cfm for the ventilation unit and 35,485 cfm ± 10% for the air conditioning unit.

2. Verifying that on a control room isolation test signal, the system automatically switches into the emergency mode of operation with flow through the HEPA filters and charcoal adsorber banks.

3. Verifying that on a toxic gas isolation test signal, the system automatically switches into the isolation mode of operation with flow through the HEPA filters and charcoal adsorber banks.

4. Verifying that the system maintains the control room at a positive pressure of greater than or equal to 1/8 inch W.G. relative to the outside atmosphere during system operation in the emergency mode.

5. Verifying that the heaters dissipate 4.8 kw ± 5% when tested in accordance with ANSI N510-1975.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Southern California Edison Company, et al. (SCE or the licensee), dated August 26, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 112 , are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

William H. Bateman for

Mel B. Fields, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 26, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by the Amendment number and contains marginal lines indicating the areas of change.

REMOVE	INSERT
3/4 7-14	3/4 7-14

3/4.7.5 CONTROL ROOM EMERGENCY AIR CLEANUP SYSTEM*

LIMITING CONDITION FOR OPERATION

3.7.5 Two independent control room emergency air cleanup systems shall be OPERABLE.

APPLICABILITY: ALL MODES or during movement of irradiated fuel assemblies.

ACTION:

Unit 2 or 3 in MODES 1, 2, 3 or 4:

With one control room emergency air cleanup system inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

Units 2 or 3 in MODES 5 or 6; or defueled when moving irradiated fuel assemblies:

a. With one control room emergency air cleanup system inoperable, restore the inoperable system to OPERABLE status within 7 days or initiate and maintain operation of the remaining OPERABLE control room emergency air cleanup system in the recirculation mode.

b. With both control room emergency air cleanup systems inoperable, or with the OPERABLE control room emergency air cleanup system required to be in the recirculation mode by ACTION (a), not capable of being powered by an OPERABLE emergency power source, suspend all operations involving CORE ALTERATIONS or positive reactivity changes, or movement of irradiated fuel assemblies.

c. The provisions of Specification 3.0.3 are not applicable in MODE 6.

d. The provisions of Specification 3.0.4 are not applicable when entering MODES 5, 6 or defueled configuration.

SURVEILLANCE REQUIREMENTS

4.7.5 Each control room emergency air cleanup system shall be demonstrated OPERABLE:

a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 110°F.

b. At least once per 31 days on a STAGGERED TEST BASIS by initating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the system operates for at least 10 hours with the heaters on.

c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the system by:

   1. Verifying that with the system operating at a flow rate of 35485 cfm ± 10% for the air conditioning unit, and 2050 ± 150 cfm for the ventilation unit and recirculating through the respective HEPA filters and charcoal adsorbers, leakage through the system diverting valves is less than or equal to 1% air conditioning unit and 1% ventilation unit when the system is tested by admitting cold DOP at the respective intake.

*Shared system with San Onofre - Unit 2.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-10

AND AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

By letter dated August 26, 1994, Southern California Edison Company, et al. (SCE or the licensee), submitted a request for changes to the Technical Specifications (TS) for San Onofre Nuclear Generating Station, Unit Nos. 2 and 3 (SONGS). The proposed changes would revise TS 3.7.5, "Control Room Emergency Air Cleanup System," to (1) add "movement of irradiated fuel assemblies" to the modes for which this limiting condition for operation (LCO) applies, and (2) provide an LCO 3.0.4 exception for Modes 5 and 6 and a defueled configuration.

2.0 EVALUATION

The LCOs in TS 3.7.5 define the operability requirements for the control room emergency air cleanup system (CREACUS) for all modes. This system provides a protected environment from which operators can control the plant following an uncontrolled release of radioactivity or toxic gas.

2.1 Proposed Changes to the APPLICABILITY and ACTION statements

TS 3.7.5 presently applies to all operational modes, but not to movement of irradiated fuel assemblies. Mode 6 (refueling) is characterized as the operational mode when fuel is in the reactor vessel. Movement of irradiated fuel inside or outside containment when no fuel is in the vessel is not covered by this definition.

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The SONGS Units 2 and 3 Updated Final Safety Analysis Report (UFSAR), Section 15.7.3.4, "Design Basis Fuel Handling Accidents," describes the consequences of a fuel-handling accident inside containment and inside the fuel-handling building. Since the fuel-handling accident is considered a limiting fault, it is postulated that a fuel assembly is dropped during refueling operations, breaching the cladding of the fuel pins and releasing the volatile fission products contained in the gap region of the fuel pin. Radiation doses to control room personnel following a postulated fuel handling accident are based on the same shielding, ventilation, atmospheric dispersion, and dose model assumptions used for the loss-of-coolant accident. Buildup of activity in the control room is based on the 2-hour release of activity following the accident. In calculating the doses, outside air intake through the control room heating ventilation, and air-conditioning system (emergency mode of operation) is assumed for the 2-hour duration of the accident.

The additional requirement proposed for the Applicability statement--that two trains of CREACUS be operable during movement of irradiated fuel--covers the consequences of a fuel-handling accident when there are no assemblies in the core, including (1) when fuel is moved in the fuel-handling building, (2) when the last fuel assembly is moved from the vessel (defueling) to the transfer canal inside containment, and (3) when the first fuel assembly is moved from the transfer canal to the vessel (fuel loading).

The proposed change would also revise the Action statement by adding the words "or defueled configuration when moving irradiated fuel assemblies" after Modes 5 or 6. These words are added for consistency with the proposed Applicability statement. This change clarifies the action to be entered if the LCO is not met in a defueled configuration during movement of irradiated fuel assemblies.

The proposed change would also revise Action b) by adding the words "or movement of irradiated fuel assemblies." Action b) requires that if both trains of CREACUS are inoperable or not capable of being powered by an operable emergency power source, the licensee must suspend all operations involving core alterations or positive reactivity changes. The proposed change clarifies that movement of irradiated fuel must also be suspended.

These changes are conservative in that they place additional requirements on the operability of CREACUS during movement of irradiated fuel. These changes are consistent with the accident analyses described in the UFSAR. Based on these considerations, the staff finds the requested changes acceptable.

Based on an oral discussion between the staff and the licensee, the applicability statement in TS 3.7.5 that currently reads, "Units 2 and 3 in MODE 5 or 6..." has been changed to read, "Units 2 or 3 in MODE 5 or 6...". This is an editorial change to clearly indicate that the ACTIONS under this statement apply whenever either unit is in Mode 5 or 6. This will avoid the possible misinterpretation that both units need to be in Modes 5 or 6 before the ACTIONS apply.

2.2 Proposed Exception to LCO 3.0.4 for TS 3.7.5

TS 3.0.4 prohibits entry into an operational mode or other specified condition unless the conditions of the LCO are met. Therefore, Mode 6 cannot be entered from a defueled configuration unless both trains of CREACUS are operable. The current TS provide critical path restrictions to refueling outages. When maintenance and surveillance activities are performed on the system during outages, the activities must be terminated for mode changes and later reestablished, adding to the time that the system is in a degraded condition. The proposed revision to add an exception to TS 3.0.4 when entering Modes 5 or 6 or a defueled configuration would allow one train of the system to remain inoperable during the mode change from defueled configuration to Mode 6, from Mode 6 to Mode 5, from Mode 5 to Mode 6, from Mode 6 to defueled configuration, and from Mode 4 to Mode 5. Allowing one train of CREACUS to be inoperable during the mode changes described above has no safety effect.

The transition from Mode 6 to the defueled configuration occurs when the last fuel assembly remaining in the core is being removed; during such an evolution, current TS 3.0.4 requires both CREACUS trains to be operable. This requirement also applies during the transition from the defueled configuration to Mode 6, *i.e.*, when there are no fuel assemblies in the core, and an irradiated fuel assembly is being moved into the core. The only credible event during these transitions is the design-basis fuel-handling accident. This fuel handling accident is the same design basis accident for evolutions involving the movement of an irradiated fuel assembly during Mode 6, to which the requirements of TS 3.7.5 apply. TS 3.7.5 requires one operable train of CREACUS for such an event. The design basis accident described above is covered by the proposed revision to the Applicability statement discussed in Section 2.1. With the addition of "or during movement of irradiated fuel assemblies" to the Applicability statement, operability of at least one train of CREACUS will be ensured prior to the start of movement of irradiated fuel.

Entering Mode 5 from Mode 6 consists of fully tightening the last reactor vessel head closure bolt. This evolution has no safety significance from the point of view of isolating the control room from a potential radiation or gas release and can be excepted from TS 3.0.4. The threshold of entering Mode 6 from Mode 5 consists of untightening at least one reactor vessel head closure bolt. This evolution has no safety significance from the point of view of isolating the control room from a radiation or toxic gas release and can be excepted from TS 3.0.4.

Entering Mode 5 from Mode 4 consists of decreasing reactor coolant system temperature to less than or equal to 200 degrees F by initiating shutdown cooling. This evolution has no safety significance from the point of view of isolating the control room from a radiation or toxic gas release.

The current TS allow one train of CREACUS to be inoperable for up to 7 days during all modes based on the ability of one train of CREACUS to provide control room isolation following design-basis accidents. The mode changes discussed above have no significance relative to the releases and no effect on the ability of CREACUS to perform its function.

Based on the considerations discussed above, the staff finds the exception to TS 3.0.4 when entering Modes 5, 6, and defueled configuration acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 55891). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Donna Skay

Date: July 26, 1995