August 10, 1988

Docket Nos.: 50-361 and 50-362

Mr. Kenneth P. Baskin
Vice President
Southern California Edison Company
2244 Walnut Grove Avenue
Post Office Box 800
Rosemead, California 91770

Mr. Gary D. Cotton
Senior Vice President
Engineering and Operations
San Diego Gas & Electric Company
101 Ash Street
Post Office Box 1831
San Diego, California 92112

Gentlemen:

SUBJECT: ISSUANCE OF AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NPF-10 AND AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NPF-15 SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3 (TACS 68425 AND 68426)

The Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 65 to Facility Operating License No. NPF-10 and Amendment No. 54 to Facility Operating License No. NPF-15 for the San Onofre Nuclear Generating Station, Units 2 and 3, located in San Diego County, California.

The amendments revise Technical Specification Section 3.1.3.4, "CEA Drop Time" to increase the allowable time for insertion of control element assemblies (CEA's) upon receipt of a reactor trip signal from 3.0 seconds to 3.2 seconds.

These amendments cover Proposed Change Number (PCN) 263 and were requested by your letter of June 14, 1988, as supplemented by letters dated July 13 and July 25, 1988. A copy of the Notice of Issuance which has been forwarded to the Office of the Federal Register for publication is also enclosed.

Also enclosed for your information is a copy of the Environmental Assessment that was published in the Federal Register on August 9, 1988 (53 FR 29971).

Sincerely,

original signed by

Donald E. Hickman, Project Manager
Project Directorate V
Division of Reactor Projects - III, IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 65 to NPF-10
2. Amendment No. 54 to NPF-15
3. Safety Evaluation
4. Notice of Issuance
5. Environmental Assessment

cc: See next page

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DHagan	ARM/LFMB

*See previous concurrence

DRSP/PD5	DRSP/PD5	*OGC	DRSP/D:PD5
JLee	DEHickman:dr	MYoung	GWKnighton
8/10/88	8/10/88	08/05/88	8/10/88

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 10, 1988

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The amendments revise Technical Specification Section 3.1.3.4, "CEA Drop Time" to increase the allowable time for insertion of control element assemblies (CEA's) upon receipt of a reactor trip signal from 3.0 seconds to 3.2 seconds.

These amendments cover Proposed Change Number (PCN) 263 and were requested by your letter of June 14, 1988, as supplemented by letters dated July 13 and July 25, 1988. A copy of the Notice of Issuance which has been forwarded to the Office of the Federal Register for publication is also enclosed.

Also enclosed for your information is a copy of the Environmental Assessment that was published in the Federal Register on August 9, 1988 (53 FR 29971).

Sincerely,

Donald E. Hickman

Donald E. Hickman, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 65 to NPF-10
2. Amendment No. 54 to NPF-15
3. Safety Evaluation
4. Notice of Issuance
5. Environmental Assessment

cc: See next page

Mr. Kenneth P. Baskin
Southern California Edison Company

San Onofre Nuclear Generating
Station, Units 2 and 3

cc:

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Resident Inspector, San Onofre NPS
c/o U. S. Nuclear Regulatory Commission
Post Office Box 4329
San Clemente, California 92672

Mr. Sherwin Harris
Resource Project Manager
Public Utilities Department
City of Riverside
City Hall
3900 Main Street
Riverside, California 92522

cc:
California State Library
Government Publications Section
Library & Courts Building
Sacramento, CA 95841
ATTN: Ms. Mary Schnell

Mayor, City of San Clemente
San Clemente, CA 92672

Chairman, Board Supervisors
San Diego County
1600 Pacific Highway, Room 335
San Diego, CA 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
Sacramento, CA 95814

Mr. Jack McGurk, Acting Chief
Radiological Health Branch
State Department of Health Services
714 P Street, Building #8
Sacramento, California 95814

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the license for San Onofre Nuclear Generating Station, Unit 2 (the facility) filed by the Southern California Edison Company (SCE) on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated June 14, 1988, as supplemented July 13 and July 25, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 65 , are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The change in Technical Specifications is to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during change over shall be minimized.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

H. Rood for

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 65

FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
3/4 1-20	3/4 1-20

CEA DROP TIME

LIMITING CONDITION FOR OPERATION

3.1.3.4 The individual full length (shutdown and regulating) CEA drop time, from a withdrawn position greater than or equal to 145 inches, shall be less than or equal to 3.2 seconds from when the electrical power is interrupted to the CEA drive mechanism until the CEA reaches its 90 percent insertion position with:

a. T_{avg} greater than or equal to 520°F, and

b. All reactor coolant pumps operating.

APPLICABILITY: MODES 1 and 2.

ACTION:

With the drop time of any full length CEA determined to exceed the above limit, be in at least HOT STANDBY within six hours.

SURVEILLANCE REQUIREMENTS

4.1.3.4 The CEA drop time of full length CEAs shall be demonstrated through measurement prior to reactor criticality:

a. For all CEAs following each removal and reinstallation of the reactor vessel head,

b. For specifically affected individuals CEAs following any maintenance on or modification to the CEA drive system which could affect the drop time of those specific CEAs, and

c. At least once per 18 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the license for San Onofre Nuclear Generating Station, Unit 3 (the facility) filed by the Southern California Edison Company (SCE) on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated June 14, 1988, as supplemented July 13 and July 25, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 54, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The change in Technical Specifications is to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during change over shall be minimized.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

H. Rood for

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 10, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove	Insert
3/4 1-20	3/4 1-20

CEA DROP TIME

LIMITING CONDITION FOR OPERATION

3.1.3.4 The individual full length (shutdown and regulating) CEA drop time, from a withdrawn position greater than or equal to 145 inches, shall be less than or equal to 3.2 seconds from when the electrical power is interrupted to the CEA drive mechanism until the CEA reaches its 90 percent insertion position with:

a. T_{avg} greater than or equal to 520°F, and

b. All reactor coolant pumps operating.

APPLICABILITY: MODES 1 and 2.

ACTION:

a. With the drop time of any full length CEA determined to exceed the above limit, be in at least HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.4 The CEA drop time of full length CEAs shall be demonstrated through measurement prior to reactor criticality:

a. For all CEAs following each removal and reinstallation of the reactor vessel head,

b. For specifically affected individuals CEAs following any maintenance on or modification to the CEA drive system which could affect the drop time of those specific CEAs, and

c. At least once per 18 months.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. NPF-10

AND AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL.

SAN ONOFRE NUCLEAR GENERATING STATION UNITS 2 AND 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

By letter dated June 14, 1988, as supplemented by letters dated July 13, and July 25, 1988, Southern California Edison Company (SCE), et al. (the licensees) submitted a request to revise Technical Specification 3/4.1.3.4 "CEA Drop Time," for the San Onofre Nuclear Generating Station (SONGS) Units 2 and 3. The change would increase the time requirement for insertion of control element assemblies (CEA's) upon receipt of a reactor trip signal from 3.0 seconds to 3.2 seconds. Specifically, this represents the time from when the electrical power is interrupted to the CEA drive mechanism until the CEA reaches its 90 percent insertion position from the fully withdrawn position.

Previously each CEA was withdrawn from the core to its full out position and dropped by opening its individual circuit breaker. Beginning with Unit 2 Cycle 4 startup, a new method of measuring CEA drop times was used in which the reactor trip breakers are the point at which power is interrupted to the CEA gripper coils rather than the individual breakers. This new method uses the reactor trip breakers and, therefore, more accurately reflects the operation of the reactor protection system during a scram.

During the Unit 2 startup tests, the CEA drop times obtained using the new method were longer than those obtained using the previous method. This has been found to be due to the fact that the circuit dissipating gripper coil stored energy has a longer time constant when tripped by the reactor trip breakers than when tripped by the individual circuit breakers. Unit 3 is currently in a refueling outage and will use the new test method during Cycle 4 startup. Since a review of previous Unit 3 CEA drop time measurements indicated that there is a potential for at least one CEA to fail to meet the 3.0 seconds requirement, the proposed change would increase the specified drop time to 3.2 seconds in order to avoid possible delay of Unit 3 restart.

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2.0 EVALUATION

The staff has reviewed SCE's reevaluation of those Chapter 15 Design Basis Accidents which could be adversely impacted by the increased CEA scram time. The reevaluation of these events incorporated the increased CEA drop times in a conservative manner. Also incorporated was a revised CEA reactivity versus position curve based on space-time neutron kinetics calculations for those events which previously used static calculations. The staff has previously approved this methodology to determine CEA scram characteristics for SONGS as well as for other Combustion Engineering plants. The reevaluation has shown that, for most events, this revised scram reactivity prediction is conservative relative to the reference analysis scram reactivity data at the crucial time in the transient (during the closest approach to a safety limit), or sufficient conservatisms exist in the analyses to offset the effects of the increased CEA drop time on the consequences of previously analyzed accidents.

For the most severe increased main steam flow event, which is the inadvertent opening of all of the turbine bypass valves at full power, a low DNBR trip is generated by the Core Protection Calculators (CPC's) to terminate thermal margin degradation. The additional 0.3 second delay before CEA motion begins would result in additional margin degradation. To compensate for this, the licensee has proposed to increase the CPC DNBR power uncertainty penalty addressable constant labeled BERR1 in the CPC algorithms to 1.005. Adjustments to the value of addressable constants by SCE, without prior NRC approval, is permitted by Technical Specification 6.8.1.j, provided the new value is within the software limit values. This is the case with this adjustment. This correction factor effectively provides a CPC low DNBR trip early enough to compensate for an increased holding coil decay time of up to 0.3 seconds.

For the uncontrolled CEA withdrawal event from a subcritical condition, an additional 0.3 second holding coil decay time results in increases in peak core power from 65% to 106% of rated thermal power and peak heat flux from 16% to 33%. However, the staff's acceptance criteria remain satisfied since minimum DNBR remains greater than 1.31 and fuel centerline melting does not occur.

The increase in CEA drop time results in an increase of less than 2% in required margin to accommodate a total loss of forced reactor coolant flow and less than 1% to accommodate the pre-trip power excursion during a steam line break. The licensee will, therefore, increase the Cycle 4 Core Operating Limit Supervisory System (COLSS) required overpower margins accordingly.

The licensee has verified that sufficient margin exists in the current TS limits when COLSS is out of service to accommodate the increased delay time so that no change to this limit is required.

3.0 CONCLUSIONS

The staff finds the proposed increase in CEA drop time acceptable based on the above evaluation of its effect on the reference safety analyses. These analyses either remain bounding or continue to satisfy the staff's acceptance criteria provided that the adjustment to the BERR1 addressable constant and the increase in COLSS margins discussed above are made. The staff recommends that the Bases for Specification 3.1.3.4 be modified to reflect these required changes.

4.0 CONTACT WITH STATE OFFICIAL

The NRC staff also advised the Chief of the Radiological Health Branch, State Department of Health Services, State of California, of the proposed determination of no significant hazards consideration. No comments were received from the State or from the public.

5.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact has been prepared and published in the Federal Register (53 FR 29971, August 9, 1988). Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of this amendment will not have a significant effect on the quality of the human environment.

6.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Kopp

Dated: August 10, 1988

U.S. NUCLEAR REGULATORY COMMISSION

SOUTHERN CALIFORNIA EDISON COMPANY

DOCKET NOS. 50-362 AND 50-362

ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering the issuance of amendments to Facility Operating License Nos. NPF-10 and NPF-15 issued to Southern California Edison Company, et al. (the licensee), for operation of the San Onofre Nuclear Generating Station, Units 2 and 3, located in San Diego County, California.

ENVIRONMENTAL ASSESSMENT

Identification of the Proposed Action

Section 3/4.1.3.4 of the Technical Specifications currently requires the Control Element Assembly (CEA) drop time, from a withdrawn position greater than or equal to 145 inches, to be less than or equal to 3.0 seconds from when electrical power is interrupted to the CEA drive mechanism until the CEA reaches its 90 percent inserted position. This drop time is required to be demonstrated through measurement prior to criticality whenever the reactor vessel head has been removed, following maintenance on the CEA drive mechanisms which could affect drop time, and every 18 months. The proposed change would increase the maximum allowable drop time from 3.0 seconds to 3.2 seconds.

Need for the Proposed Action

Previously each CEA was withdrawn from the core to its full out position and dropped by opening its individual circuit breaker. Beginning with Unit 2

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Cycle 4 startup, a new method of measuring CEA drop times was used in which the reactor trip breakers are the point at which power is interrupted to the CEA gripper coils rather than the individual breakers. This new method uses the reactor trip breakers and, therefore, more accurately reflects the operation of the reactor protection system during a scram. During the Unit 2 startup tests, the CEA drop times obtained using the new method were longer than those obtained using the previous method. This has been found to be due to the fact that the circuit dissipating gripper coil stored energy has a longer time constant when tripped by the reactor trip breakers than when tripped by the individual circuit beakers. Unit 3 is currently in a refueling outage and will use the new test method during Cycle 4 startup. Since a review of previous Unit 3 CEA drop time measurements indicated that there is a potential for at least one CEA to fail to meet the 3.0 seconds requirement, the proposed change would increase the specified drop time to 3.2 seconds in order to avoid possible delay of Unit 3 restart.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed revision to the Technical Specifications and has concluded that the proposed change provides reasonable assurance that the facility can be operated safely. This change will not alter the impacts of normal operation. The proposed change does not increase the probability or consequences of accidents. There will be no increase in the probability of any accident because the CEA drive mechanisms have not been changed. With respect to the consequences of accidents, the increased drop time is either bounded by previous analyses or is compensated for by penalty factors or design conservatisms. Therefore, the consequences of

previously analyzed accidents will not be increased. No changes are being made in the types of any effluents that may be released offsite and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that this proposed action would result in no significant adverse radiological environmental impact.

With regard to potential nonradiological impacts, the proposed change to the Technical Specifications involves systems located within the restricted areas as defined in 10 CFR Part 20. It does not affect nonradiological plant effluents and has no other environmental impact. Therefore, the Commission concludes that there are no significant nonradiological environmental impacts associated with the proposed amendments.

The Notice of Consideration of Issuance of Amendments and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on July 8, 1988 (53 FR 25711). No request for hearing or petition for leave to intervene was filed following these notices.

Alternative to the Proposed Action

Since the Commission concluded that there is no significant adverse environmental effect that would result from the proposed action, alternatives with equal or greater environmental impacts need not be evaluated.

The principal alternative would be to deny the requested amendments. Denial of the request would not reduce environmental impacts of plant operation.

Alternative Use of Resources

This action does not involve the use of resources not previously considered in the Final Environmental Statement Related to the Operation of the San Onofre Nuclear Generating Station, Units 2 and 3, dated April 1981.

Agencies and Persons Consulted

The NRC staff has reviewed the licensee's request and did not consult other agencies or persons.

FINDING OF NO SIGNIFICANT IMPACT

The Commission has determined not to prepare an environmental impact statement for the proposed license amendment.

Based upon this environmental assessment, we conclude that the proposed action will not have a significant adverse effect on the quality of the human environment.

For further details with respect to this action, see the application for amendments dated June 14, 1988 and the supplementary information provided by letters dated July 13 and July 25, 1988, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the General Library, University of California at Irvine, Irvine, California 92713.

Dated at Rockville, Maryland, this 4th day of August 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

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UNITED STATES NUCLEAR REGULATORY COMMISSION

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL.

DOCKET NOS. 50-361 AND 50-362

NOTICE OF ISSUANCE OF AMENDMENTS TO

FACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 65 to Facility Operating License No. NPF-10 and Amendment No. 54 to Facility Operating License No. NPF-15, issued to Southern California Edison Company, San Diego Gas and Electric Company, The City of Riverside, California and The City of Anaheim, California (the licensees), which revised the Technical Specifications for operation of the San Onofre Nuclear Generating Station, Units 2 and 3, located in San Diego County, California.

The amendments were effective as of the date of issuance.

These amendments revise Technical Specification Section 3.1.3.4, "CEA Drop Time" in response to an application for amendments designated as PCN-263.

The application for amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on July 8, 1988 (53 FR 25711). No request for a hearing or petition for leave to intervene was filed following this notice.

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The Commission has prepared an Environmental Assessment related to the action and has determined that an environmental impact statement will not be prepared and that issuance of the amendments will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the applications for amendments dated June 14, 1988 and supplemental letters dated July 13 and July 25, 1988, (2) Amendment No. 65 to License No. NPF-10 and Amendment No. 54 License No. NPF-15, (3) the Commission's related Safety Evaluation and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N.W., and at the General Library, University of California, P.O. Box 19557, Irvine, California 92713. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects III, IV, V and Special Projects.

Dated at Rockville, Maryland this 10th day of August 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

D. E. Hickman, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation