

July 17, 2002

Mr. J. W. Moyer, Vice President
Carolina Power & Light Company
H. B. Robinson Steam Electric Plant,
Unit No. 2
3581 West Entrance Road
Hartsville, South Carolina 29550

SUBJECT: H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 - REQUEST FOR
ADDITIONAL INFORMATION (RAI) ON AMENDMENT APPLICATION
REGARDING TECHNICAL SPECIFICATION CHANGES TO INCREASE
AUTHORIZED POWER LEVEL (TAC NO. MB5106)

Dear Mr. Moyer:

By letter dated May 16, 2002, you submitted an amendment application regarding technical specification changes to increase the authorized power level for the H. B. Robinson Steam Electric Plant, Unit No. 2. The NRC staff reviewed the information that you provided in the subject amendment application, needed additional information to complete our review, and sent you an RAI by letter dated June 26, 2002. Upon further review, our Mechanical and Civil Engineering staff identified the need for further information specific to the Mechanical and Civil items in your subject application, as described in the enclosed RAI.

This request for information was discussed with Mr. Curt Castell and your staff on July 12, 2002, and a mutually agreeable schedule for a response by July 22, 2002, was established for your comprehensive response to both RAIs. We regret that we did not foresee this additional information need earlier.

If you have any questions, please contact me on (301) 415 1478.

Sincerely,

/RA/

Ram Subbaratnam, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-261

Enclosure: RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
REQUEST FOR TECHNICAL SPECIFICATIONS CHANGES TO
INCREASE AUTHORIZED REACTOR CORE POWER LEVEL
TAC NO. MB5106

Carolina Power & Light Company is requested to provide the following additional information, relating to Attachment II to application RNP-RA/02-0066, in support of the staff's review of the May 16, 2002, request for Technical Specifications changes to increase the authorized reactor thermal power level from 2300 megawatts to 2339 megawatts for H. B. Robinson Steam Electric Plant, Unit No. 2.

1. Section 3.6.2 provides an assessment of reactor coolant system (RCS) components for the uprated power conditions. The reactor pressure vessel was not explicitly addressed. The licensee is requested to provide information (i.e., existing minimum margin in stress and cumulative usage factor) to show that the existing margins for the reactor vessel will accommodate the RCS temperature change or that the current design/operating temperature range is bounding for the 1.7% uprate condition.
2. Section 3.6.2.1 states that, "the values for T_{hot} and T_{cold} under uprated power conditions are bounded by the **revised RCS design condition** values as shown in Table 3.6-1." Temperature data in Table 3.6-1 indicates that the T_{hot} for the revised RCS design condition (604.6°F) is slightly higher than that of the uprated operating conditions (604.1°F or 604.5°F). However, the T_{cold} for the revised RCS design conditions (546.1°F) is slightly lower than that of the uprated operating conditions (547.6°F or 547.3°F). On the basis of the cited temperatures from Table 3-6.1, provide your justification for the above quoted statement.
3. Section 3.6.2.8 concluded that primary water stress-corrosion cracking of the control rod drive mechanism (CRDM) nozzles will not be impacted by the slight increase in RCS T_{hot} resulting from the power uprate. It further stated that an inspection of the CRDM nozzles is planned for the next refueling outage (RO-21) scheduled to begin in October 2002. Please provide your clarification regarding the significance of the planned inspection as it relates to its impact on the Technical Specification amendment request for power uprate.
4. Per the guidance provided in NRC Regulatory Issue Summary 2002-03: "Guidance on the Content of Measurement Uncertainty Recapture Power Uprate Applications," Attachment 1, Section IV. C., provide a response that addresses any effects of the proposed power uprate on:
 - a) the pressurized thermal shock evaluation for the H. B. Robinson reactor pressure vessel materials, which should demonstrate continued compliance with the requirements in Title 10 of the Code of Federal Regulations Section 50.61, and
 - b) the upper shelf energy evaluation for the H. B. Robinson reactor pressure vessel materials, which should demonstrate continued compliance with the requirements in Title 10 of the Code of Federal Regulations Part 50, Appendix G.

If the proposed power uprate has no effect on these evaluations, please state so in your response.

Mr. J. W. Moyer
Carolina Power & Light Company

H. B. Robinson Steam Electric
Plant, Unit No. 2

cc:

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