

March 1, 1999

Mr. Michael B. Sellman
Senior Vice President and
Chief Nuclear Officer
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, WI 53201

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: CLARIFICATION OF TECHNICAL SPECIFICATION
TABLE NOTATION DEFINITION (TAC NOS. MA4325 AND MA4326)

Dear Mr. Sellman:

The Commission has issued the enclosed Amendment No. 186 to Facility Operating License No. DPR-24 and Amendment No. 191 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated September 28, 1998.

These amendments clarify the notation definition of refueling interval, "R," in TS Table 15.4.1-1, "Minimum Frequencies for Checks, Calibrations, and Tests of Instrument Channels," and add a new annual interval, "A."

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Original signed by:

Beth A. Wetzel, Senior Project Manager
Project Directorate III-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: 1. Amendment No. 186 to DPR-24
2. Amendment No. 191 to DPR-27
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Beth A. Wetzel", is positioned above the typed name.

Beth A. Wetzel, Senior Project Manager
Project Directorate III-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

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cc w/encls: See next page

Mr. Michael B. Sellman
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Units 1 and 2

cc:

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Two Rivers, Wisconsin 54241

DATED: March 1, 1999

AMENDMENT NO. 186 TO FACILITY OPERATING LICENSE NO. DPR-24 - POINT BEACH UNIT 1
AMENDMENT NO. 191 TO FACILITY OPERATING LICENSE NO. DPR-27 - POINT BEACH UNIT 2

Docket File (50-266, 50-301)

PUBLIC

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**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 28, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

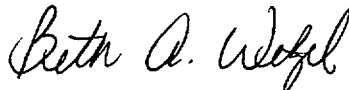
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186, are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Senior Project Manager
Project Directorate III-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: March 1, 1999



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 191
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 28, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 191, are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Senior Project Manager
Project Directorate III-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: March 1, 1999

ATTACHMENT TO LICENSE AMENDMENT NO.186
TO FACILITY OPERATING LICENSE NO. DPR-24
AND LICENSE AMENDMENT NO. 191
TO FACILITY OPERATING LICENSE NO. DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

Table 15.4.1-1 (Pages 3-5)

INSERT

Table 15.4.1-1 (Pages 3-5)

TABLE 15.4.1-1 (continued)

<u>NO.</u>	<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>TEST</u>	<u>PLANT CONDITIONS WHEN REQUIRED</u>
20.	Auxiliary Feedwater Flowrate	(13)	R	-	ALL
21.	Boric Acid Control System	-	R	-	ALL
22.	Boric Acid Tank Level	D	R	-	ALL
23.	Charging Flow	-	R	-	ALL
24.	Condensate Storage Tank Level	S(1)	R	-	ALL
25.	Containment High Range Radiation	M(1)	R(14)	-	ALL
26.	Containment Hydrogen Monitor	D	-	-	ALL
	-Gas Calibration	-	Q(15)	-	ALL
	-Electronic Calibration	-	R	-	ALL
27.	Containment Pressure	S	R	Q(1,3,9)	ALL
28.	Containment Water Level	M	R	-	ALL
29.	Emergency Plan Radiation Survey Instruments	Q	A	Q	ALL
30.	Environmental Monitors	M	-	-	ALL
31.	In-Core Thermocouples	M	R(14)	-	ALL
32.	Low Temperature Overpressure Protection System	S(12)	R	(10)	ALL
33.	PORV Block Valve Position Indicator	Q	R	-	ALL
34.	PORV Operability	-	R	Q(11)	ALL
35.	PORV Position Indicator	S(21)	R	R	ALL

TABLE 15.4.1-1 (continued)

PLANT CONDITIONS
WHEN REQUIRED

<u>NO.</u>	<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>TEST</u>	
36.	Radiation Monitoring System				
	- RE-218 WDS Liquid Monitor	(7)	R(14)	Q	ALL
	- RE-223 Waste Distillate Overboard Monitor	(7)	R(14)	Q	ALL
	- RE-231 A Steam Line Release Monitor	M(1)	R(14)	-	ALL
	- RE-232 B Steam Line Release Monitor	M(1)	R(14)	-	ALL
	- RE-101 Control Room Monitor	S	R(14)	Q	ALL
	- RE-235 Control Room Noble Gas Monitor	S	R(14)	Q	ALL
	- RE-215 Air Ejector Monitor	D(1)	R(14)	-	ALL
37.	Reactor Vessel Fluid Level System	M	R	-	ALL
38.	Refueling Water Storage Tank Level	-	R	-	ALL
39.	Residual Heat Removal Pump Flow	-	R	-	ALL
40.	Safety Valve Position Indicator	M	R	-	ALL
41.	Subcooling Margin Monitor	M	R	-	ALL
42.	Deleted				
43.	Volume Control Tank Level	-	A	-	ALL
44.	Reactor Protection System and Emergency Safety Feature Actuation System Logic	-	-	M(1,23)	ALL
45.	Reactor Trip System Interlocks				
	-Intermediate Range Neutron Flux, P-6	-	R(24)	R	ALL
	-Power Range Neutron Flux, P-8	-	R(24)	R	ALL
	-Power Range Neutron Flux, P-9	-	R(24)	R	ALL
	-Power Range Neutron Flux, P-10	-	R(24)	R	ALL
	-1st Stage Turbine Impulse Pressure	-	R(24)	R	ALL

NOTATION USED IN TABLE 15.4.1-1

- A-Annually (12 months)
- S- Each shift
- D- Daily
- W- Weekly
- Q- Quarterly
- M- Monthly
- P- Prior to reactor criticality if not performed during the previous week.
- R- Each refueling interval (18 months)
- PWR- Power and Low Power Operation, as defined in Specifications 15.1.h. and 15.1.m.
- HOT S/D- Hot Shutdown, as defined in Specification 15.1.g.1.
- COLD S/D- Cold Shutdown, as defined in Specification 15.1.g.2.
- REF S/D- Refueling Shutdown, as defined in Specification 15.1.g.3.
- ALL- All conditions of operation, as defined in Specifications 15.1.g, h, and m.

NOTES USED IN TABLE 15.4.1-1

- (1) Not required during periods of refueling shutdown, but must be performed prior to reactor criticality if it has not been performed during the previous surveillance period.
- (2) Tests of the low power trip bistable setpoints which cannot be done during power operations shall be conducted prior to reactor criticality if not done in the previous surveillance interval.
- (3) Perform test of the isolation valve signal.
- (4) Perform by means of the moveable incore detector system.
- (5) Recalibrate if the absolute difference is ≥ 3 percent.
- (6) Verification of proper breaker alignment and that the 120 Vac instrument buses are energized.
- (7) Source check is required prior to initiation of a release. Source check is an assessment of channel response by exposing the detector to a source of increased radiation. Channel check is required shiftily during a release. If monitor or isolation function is discovered inoperable, discontinue release immediately.
- (8) Verify that the associated rod insertion limit is not being violated at least once per 4 hours whenever the rod insertion limit alarm for a control bank is inoperable.
- (9) Test of Narrow Range Pressure, 3.0 psig, -3.0 psig excluded.



**UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 186

TO FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 191 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated September 28, 1998, the Wisconsin Electric Power Company (the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License No. DPR-24 for Point Beach Nuclear Plant, Unit 1, and Facility Operating License No. DPR-27 for Point Beach Nuclear Plant, Unit 2. The proposed amendments would clarify the notation definition of "R" in TS Table 15.4.1-1, "Minimum Frequencies for Checks, Calibrations, and Tests of Instrument Channels," and add a new frequency "A."

2.0 EVALUATION

2.1 Background

The proposed change would involve a revision to the notation "R" used in various columns in TS Table 15.4.1-1 to specify a required surveillance frequency. The notation "R" is currently defined as a frequency of "Each refueling interval (but not to exceed 18 months)." The proposed change would redefine "R" as a frequency of "Each refueling interval (18 months)," consistent with NUREG 1431, Rev.1, "Standard Technical Specifications, Westinghouse Plants," (STS). In addition, a new notation "A" is proposed to be used to designate an annual 12-month frequency and would be defined as "Annually (12 months)" for instrumentation surveillance frequencies remaining on an annual frequency. This new frequency, "A," would replace the current calibration frequency of "R" for TS Table 15.4.1-1 Item 29, "Emergency Plan Radiation Survey Instruments," and Item 43, "Volume Control Tank Level."

The proposed changes are necessary to clarify that the provisions of TS 15.4.0.2, which allows up to a 25% extension of surveillance frequency, apply to the "R" and "A" frequency requirements. The proposed changes are consistent with STS Section SR (surveillance requirement) 3.0.2, which states that, "The specified Frequency for each SR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met. . . . If a Completion Time requires periodic performance on a "once

per . . . basis, the above Frequency extension applies to each performance after the initial performance. . . ." The changes are necessary because the Unit 1 operating cycle (cycle 25) is currently scheduled for 15 months, which would allow Unit 1 to operate until October 1999. Without application of the 25% extension interval, the TS Table 15.4.1-1 required surveillances for Unit 1 would fall due in August 1999, since the TS-required surveillances were last performed in February 1998. Wisconsin Electric would like to avoid an unnecessary plant shutdown in the peak summer month of August 1999 to perform the currently required surveillances. In addition, Wisconsin Electric is currently planning to implement 18-month cycles for both Point Beach units, and would like the proposed change to clarify that the refueling interval is 18 months and that the provisions of TS 15.4.0.2 apply to the frequencies specified in TS Table 15.4.1-1.

2.2 Evaluation of Proposed Changes

The proposed change to the definition of notation "R" in TS Table 15.4.1-1 is intended to clarify that the refueling interval is 18 months and that the provisions of TS 15.4.0.2, which allows a 25% extension of surveillance frequency, can be applied, and is consistent with the wording and allowances contained in STS. The licensee conducted instrument drift studies for calibration of instruments contained in TS Table 15.4.1-1 that perform safety functions, including providing the capability for safe shutdown. The purpose of these studies was to ensure appropriate safety limits and functions would be met if the 25% extension of surveillance interval (18 months plus the TS 15.4.0.2 allowable 25%, or 22.5 months) was applied to the instrumentation contained in TS Table 15.4.1-1. The drift studies and corresponding calculations determined that the magnitude of the instrument drift, for instrumentation affected by drift, that could occur over a 22.5-month interval was bounded by the uncertainty allowances used in determining safety system setpoints.

In addition, calibration data from surveillance records were reviewed for the instrumentation contained in TS Table 15.4.1-1. The purpose of this review was to determine the impact that an increase of 25% (i.e., to a total of 22.5 months) in the surveillance frequency would have on instrument availability. This review identified that as-found and as-left data has not exceeded acceptable limits for the calibration intervals except on rare occasions. Because of the very small percentage of failures that are detected on the current surveillance intervals and because of system redundancy, Point Beach has concluded that the change in the surveillance frequency will have a small impact, if any, on system availability, with the exception of those items noted below.

The new notation "A" proposed to be used to designate an annual 12-month frequency was necessary because the licensee's surveillance record review identified that instrumentation included in TS Table 15.4.1-1 Item 43, "Volume Control Tank Level," and Item 29, "Emergency Plan Radiation Survey Instruments," would not support a possible surveillance interval up to 22.5 months. As a result, Wisconsin Electric determined that these items should remain on their current annual calibration frequency, and has introduced the new notation "A" to incorporate this requirement. Since Point Beach has historically operated on a nominal 12-month cycle, the annual frequency is consistent with past operations.

A program to monitor instrumentation and control preventive maintenance, corrective maintenance, and surveillance test histories is in place at Point Beach (nuclear procedure NP 8.3.5, "Machinery History - Instrumentation and Control"). The program currently requires a

semiannual and annual review of instrumentation and control histories important to safety. The intent of this program is to identify any adverse trends in instrument and control machinery performance and ensure appropriate corrective actions are implemented.

The use of the allowance to extend surveillance intervals by 25% can also result in a significant safety benefit for surveillances that are performed on a routine basis during plant operation. This safety benefit is incurred when a surveillance interval is extended at a time that conditions are not suitable for performing the surveillance. Examples of this include transient plant operating conditions or conditions in which safety systems are out of service because of ongoing surveillance or maintenance activities. In such cases, the safety benefit of allowing the use of the 25% allowance to extend a surveillance interval would likely outweigh any benefit derived by limiting the intervals to the presently interpretable 18 months. The limitation of TS 15.4.0.2 is based on engineering judgment and the recognition that the most probable result of any surveillance being performed is the verification of conformance with the surveillance requirements. This provision is sufficient to ensure that the reliability ensured through surveillance activities is not significantly degraded beyond that obtained from the specified surveillance interval.

The proposed change to the definition of the "R" notation for TS Table 15.4.1-1 clarifies that the refueling interval is 18 months and that the 25% surveillance frequency extension allowed by TS 15.4.0.2 can be applied. The change was supported by licensee instrument drift studies and surveillance record reviews and is consistent with STS. Instrumentation that would not support the surveillance frequency extension was retained at an annual surveillance frequency, consistent with past plant operations. Therefore, the proposed change is acceptable to the staff.

The proposed addition of the "A" notation to TS Table 15.4.1-1 and the change in calibration frequency for Item 29, "Emergency Plan Radiation Survey Instruments," and Item 43, "Volume Control Tank Level," from "R" to "A" is consistent with past plant operations and is administrative. Therefore, the proposed changes are acceptable to the staff.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (64 FR 4162). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Lyon

Date: March 1, 1999

March 1, 1999

Mr. Michael B. Sellman
Senior Vice President and
Chief Nuclear Officer
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, WI 53201

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
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Original signed by:

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Project Directorate III-1
Division of Licensing Project Management
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Docket Nos. 50-266 and 50-301

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