

October 24, 1997

Mr. Richard R. Grigg
Chief Nuclear Officer
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2 - CORRECTION TO
AMENDMENT NOS. 181 AND 185 RE: CONTAINMENT INTEGRATED LEAK
RATE TESTING (TAC NOS. M99376 AND M99377)

Dear Mr. Grigg:

On September 29, 1997, the Commission issued Amendments 181 and 185 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments revised the technical specifications (TS) to incorporate the provisions of 10 CFR Part 50, Appendix J, Option B.

The amendments for both Units 1 and 2 were issued with a typographical error on page 15.4.2-5, in that TS 15.4.2.B.1 references 10 CFR 50, Section 50.55(g)(6)(i) when the correct reference should be 10 CFR 50, Section 50.55a(g)(6)(i). Also, page 15.6.12-1, TS 15.6.12.D.1., was corrected to insert the word "rate" that had been inadvertently omitted.

We regret any inconvenience these oversights may have caused. If you have any questions on this action, please call me at 301 415-1380.

Sincerely,

ORIGINAL SIGNED BY

Linda L. Gundrum, Project Manager
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Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: 1. Corrected TS Page 15.4.2-5
2. Corrected TS Page 15.6.12-1

cc w/encls: See next page

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DATE	10/7/97		10/7/97		10/10/97		10/24/97	

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Mr. Richard R. Grigg
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

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B. In-Service Inspection and Testing of Safety Class Components Other than Steam Generator Tubes

1. Inservice inspection of ASME Code Class 1, Class 2 and Class 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g) modified by Section 50.55a(b), except where specific written relief is granted by the NRC, pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
 - a. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.
2. Containment isolation valves will be tested in accordance with the Containment Leakage Rate Testing Program.
3. Inservice testing of ASME Code Class 1,2, and 3 pumps, valves, and snubbers shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a.
 - a. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

Basis

The steam generator tube inspection requirements are based on the guidance given in NRC Regulatory Guide 1.83, "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes." ASME Section XI Appendix IV is being used for defining the basic requirements or the inspection method. However, at the present time, changes and improvements in steam generator eddy current inspection are occurring faster than the code can be revised. Thus, in order to ensure that the best possible exam of the tubing and/or sleeves is being done, the technique utilized will, in general, be the latest industry-accepted technique. This means that complete word-for-word compliance with Appendix IV may not be possible. However, the basic requirements and intent will be met, to the extent practical.

Specification 15.4.2.B delineates programmatic requirements for establishing Inservice Inspection and Testing programs in accordance with the ASME Section XI Code and 10 CFR 50.55a requirements. The Code establishes criteria for system and component inspection and testing to ensure an appropriate level of reliability and detection of abnormal conditions. Failure to meet Code requirements is evaluated on an individual system or component bases to determine operability. Appropriate LCOs are entered if a system or component is determined to be inoperable.

As stated in 15.4.2.B.1, safety class components, other than the steam generator tubing, will be inspected in accordance with ASME Section XI. The code edition/addenda utilized for the inspection interval will be as defined in

Unit 1 - Amendment No. 63, 95, 450, 181

Unit 2 - Amendment No. 68, 99, 454, 185

Corrected October 24 , 1997

15.4.2-5

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CONTAINMENT LEAKAGE RATE TESTING PROGRAM

- A. A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September, 1995.
- B. The peak design containment internal accident pressure, P_a is 60 psig.
- C. The maximum allowable primary containment leakage rate, L_a at P_a , shall be 0.4% of containment air weight per day.
- D. Leakage rate acceptance criteria are:
 - 1. The containment leakage rate acceptance criterion is $\leq 1.0 L_a$.
 - 2. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.6 L_a$ for the combined Type B and Type C tests and $\leq 0.75 L_a$ for Type A tests.
- E. The provisions of Specification 15.4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.
- F. The provisions of Specification 15.4.0.3 are applicable to the Containment Leakage Rate Testing Program.

DATED: October 24, 1997

CORRECTION LETTER TO AMENDMENT NOS. 181 AND 185 TO FACILITY OPERATING
LICENSE NOS. DPR-24 AND DPR-27 - POINT BEACH UNITS 1 AND 2

Docket File

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