

Mr. Michael B. Sellman
 Chief Nuclear Officer
 Wisconsin Electric Power Company
 231 West Michigan Street
 Milwaukee, WI 53201

July 13, 1998

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS RE: RELOCATION OF RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATIONS (TAC NOS. M95350 AND M95351)

Dear Mr. Sellman:

The Commission has issued the enclosed Amendment Nos. 184 and 188 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated April 24, 1996, as supplemented on December 15, 1997, and June 22, 1998 (TSCR-174).

These amendments revise TS Section 15.7, "Radiological Effluent Technical Specifications (RETS)." Sections 15.7.3, 15.3.4, 15.7 of RETS were moved to licensee-controlled documents consistent with Generic Letter 89-01, "Implementation of Programmatic Controls for Radiological Effluent Technical Specifications in the Administrative Controls Section of the Technical Specifications and the Relocation of Procedural Details of RETS to the Offsite Dose Calculation Manual or to the Process Control Program." Other sections of the TS were revised consistent with the removal of portions of the RETS. Additionally, explosive gas monitoring instrumentation requirements were relocated to the Radiological Effluent Control Program, explosive gas concentration limits remain in TS 15.7.5, and TS 15.7.8 revised to require that any revisions to explosive gas requirements be evaluated in accordance with 10 CFR 50.59. These changes are in accordance with Generic Letter 95-10, "Relocation of Selected Technical Specifications Requirements Related to Instrumentation."

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY
 Linda L. Gundrum, Project Manager
 Project Directorate III-1
 Division of Reactor Projects - III/IV
 Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

- Enclosures: 1. Amendment No. 184 to DPR-24
 2. Amendment No. 188 to DPR-27
 3. Safety Evaluation

cc w/encls: See next page

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DATED: July 13, 1998

AMENDMENT NO. 184 TO FACILITY OPERATING LICENSE NO. DPR-24 - POINT BEACH UNIT 1
AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-27 - POINT BEACH UNIT 2

Docket File (50-266, 50-301)

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March 1998



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 184
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 24, 1996, as supplemented December 15, 1997, and June 22, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended to approve the relocation of certain Technical Specification requirements to licensee-controlled documents, as described in the licensee's application dated April 24, 1996, as supplemented by letter dated December 15, 1997, and June 22, 1998, and evaluated in the staff's safety evaluation dated July 13, 1998. This license is also hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 184, are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance. Implementation shall also include relocation of certain Technical Specification requirements to licensee-controlled documents, as described in the licensee's application dated April 24, 1996, as supplemented by letter dated December 15, 1997, and June 22, 1998, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: July 13, 1998



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 188
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 24, 1996, as supplemented on December 15, 1997, and June 22, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended to approve the relocation of certain Technical Specification requirements to licensee-controlled documents, as described in the licensee's application dated April 24, 1996, as supplemented by letter dated December 15, 1997, and June 22, 1998, and evaluated in the staff's safety evaluation dated July 13, 1998. This license is also hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 188, are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance. Implementation shall also include relocation of certain Technical Specification requirements to licensee-controlled documents, as described in the licensee's application dated April 24, 1996, as supplemented by letter dated December 15, 1997, and June 22, 1998, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: July 13, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 184

TO FACILITY OPERATING LICENSE NO. DPR-24

AND LICENSE AMENDMENT NO. 188

TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

INSERT

TS 15-i
TS 15-ii

TS 15-i
TS 15-ii

TS 15.3.9-1

TS 15.3.9-1

TS Table 15.4.1-1 (page 5 of 6)
TS 15.4.10-1

TS Table 15.4.1-1 (page 5 of 6)
TS 15.4.10-1

TS 15.6.5-3

TS 15.6.5-3

TS 15.7.1-1 thru TS 15.7.1-3
TS 15.7.2-1
TS Figure 15.7.1-1
TS 15.7.3-1 thru 15.7.3-7
TS 15.7.4-1
TS Table 15.7.4-1
TS Table 15.7.4-2 (2 pages)
Notations for Tables 15.7.4-1
and 15.7.4-2

TS 15.7.1-1
TS 15.7.2-1
TS 15.7.2-2 (Figure 15.7.2-1)
TS 15.7.3-1
TS 15.7.4-1
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TS 15.7.5-1 thru TS 15.7.5-8
TS 15.7.6-1

TS 15.7.5-1
TS 15.7.6-1

TS Table 15.7.6-1
Notes for Table 15.7.6-1

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TS Table 15.7.6-2
Notes for Table 15.7.6-2

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TS 15.7.7-1 thru TS 15.7.7-4
TS Table 15.7.7-1 (2 pages)

TS 15.7.7-1
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TS Table 15.7.7-2
Notes for Table 15.7.7-2

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TS Table 15.7.7-3
TS 15.7.8-1 thru TS 15.7.8-6

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TS 15.7.8-1 thru 15.7.8-7

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
15	TECHNICAL SPECIFICATIONS AND BASES	
15.1	Definitions	15.1-1
15.2.0	Safety Limits and Limiting Safety System Settings	15.2.1-1
15.2.1	Safety Limit, Reactor Core	15.2.1-1
15.2.2	Safety Limit, Reactor Coolant System Pressure	15.2.2-1
15.2.3	Limiting Safety System Settings, Protective Instrumentation	15.2.3-1
15.3	Limiting Conditions for Operation	15.3.0-1
15.3.0	General Consideration	15.3.0-1
15.3.1	Reactor Coolant System	15.3.1-1
15.3.2	Chemical and Volume Control System	15.3.2-1
15.3.3	Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray	15.3.3-1
15.3.4	Steam and Power Conversion System	15.3.4-1
15.3.5	Instrumentation System	15.3.5-1
15.3.6	Containment System	15.3.6-1
15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
15.3.13	Shock Suppressors (Snubbers)	15.3.13-1
15.3.14	Fire Protection System (This section was deleted as of 1/8/97)	15.3.14-1
15.3.15	Low Temperature Overpressure Protection System	15.3.15-1
15.3.16	Reactor Coolant System Pressure Isolation Valves	15.3.16-1
15.4	Surveillance Requirements	15.4-1
15.4.1	Operational Safety Review	15.4.1-1
15.4.2	In-Service Inspection of Safety Class Components	15.4.2-1
15.4.3	Primary System Testing Following Opening	15.4.3-1
15.4.4	Containment Tests	15.4.4-1
15.4.5	Emergency Core Cooling System and Containment Cooling System Tests	15.4.5-1
15.4.6	Emergency Power System Periodic Tests	15.4.6-1
15.4.7	Main Steam Stop Valves	15.4.7-1
15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Radiological Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.4.12	Miscellaneous Radioactive Materials Sources	15.4.12-1
15.4.13	Shock Suppressors (Snubbers)	15.4.13-1
15.4.14	Surveillance of Auxiliary Building Crane Lifting Devices	15.4.14-1
15.4.15	Fire Protection System (This section was deleted as of 1/8/97)	15.4.15-1
15.4.16	Reactor Coolant System Pressure Isolation Valves Leakage Tests	15.4.16-1

TABLE OF CONTENTS (Continued)

<u>Section</u>	<u>Title</u>	<u>Page</u>
15.5	Design Features	15.5.1-1
15.5.1	Site	15.5.1-1
15.5.2	Containment	15.5.2-1
15.5.3	Reactor	15.5.3-1
15.5.4	Fuel Storage	15.5.4-1
15.6	Administrative Controls	15.6.1/2-1
15.6.1	Responsibility	15.6.1/2-1
15.6.2	Organization	15.6.1/2-1
15.6.3	Facility Staff Qualifications	15.6.3/4/5-1
15.6.4	Training	15.6.3/4/5-1
15.6.5	Review and Audit	15.6.3/4/5-1
15.6.6	Reportable Event Action	15.6.6-1
15.6.7	Action To Be Taken If A Safety Limit Is Exceeded	15.6.7-1
15.6.8	Plant Operating Procedures	15.6.8-1
15.6.9	Plant Reporting Requirements	15.6.9-1
15.6.10	Plant Operating Records	15.6.10-1
15.6.11	Radiation Protection Program	15.6.11-1
15.7	Radiological Effluent Technical Specifications (RETS)	
15.7.1	Definitions	15.7.1-1
15.7.2	Site Description	15.7.2-1
15.7.3	Radioactive Effluent Monitoring Instrumentation Operability Requirements	15.7.3-1
15.7.4	Radioactive Effluent Monitoring Instrumentation Surveillance Requirements	15.7.4-1
15.7.5	Radioactive Effluent Release and Explosive Gas Concentration Limits	15.7.5-1
15.7.6	Radioactive Effluent Sampling and Analysis Requirements	15.7.6-1
15.7.7	Operational Environmental Monitoring Program	15.7.7-1
15.7.8	Administrative Controls	15.7.8-1

15.3.9

Effluent Release

Radioactive Effluent Release limits, effluent sampling, and effluent analyses requirements are contained in the Radiological Effluent and Materials Control and Accountability Program Manual.

15.3.9-1

Unit 1 - Amendment No. 97, 184
Unit 2 - Amendment No. 404, 188

NOTATION USED IN TABLE 15.4.1-1

S- Each shift

D- Daily

W- Weekly

Q- Quarterly

M- Monthly

P- Prior to reactor criticality if not performed during the previous week.

R- Each refueling interval (but not to exceed 18 months)

PWR- Power and Low Power Operation, as defined in Specifications 15.1.h. and 15.1.n.

HOT S/D- Hot Shutdown, as defined in Specification 15.1.g.1

COLD S/D- Cold Shutdown, as defined in Specification 15.1.g.2

REF S/D- Refueling Shutdown, as defined in Specification 15.1.g.3.

ALL- All conditions of operation, as defined in Specifications 15.1.g, h and n.

NOTES USED IN TABLE 15.4.1-1

- (1) Not required during periods of refueling shutdown, but must be performed prior to reactor criticality if it has not been performed during the previous surveillance period.
- (2) Tests of the low power trip bistable setpoints which cannot be done during power operations shall be conducted prior to reactor criticality if not done in the previous surveillance interval.
- (3) Perform test of the isolation valve signal.
- (4) Perform by means of the moveable incore detector system.
- (5) Recalibrate if the absolute difference is ≥ 3 percent.
- (6) Verification of proper breaker alignment and that the 120 Vac instrument buses are energized.
- (7) Radioactive Effluent Monitoring Instrumentation Surveillance Requirements are specified in the Radiological Effluent and Materials Control and Accountability Manual.
- (8) Verify that the associated rod insertion limit is not being violated at least once per 4 hours whenever the rod insertion limit alarm for a control bank is inoperable.
- (9) Test of Narrow Range Pressure, 3.0 psig, -3.0 psig excluded.

15.4.10

RADIOLOGICAL ENVIRONMENTAL MONITORING

The Radiological Environmental Monitoring Program (REMP) sampling and analyses requirements are addressed in the Radiological Effluent and Materials Control and Accountability Program Manual. The requirement for the REMP is specified in Section 15.7.8.3.

15.4.10-1

Unit 1 - Amendment 20, 97, 184
Unit 2 - Amendment 25, 404, 188

15.6.5 REVIEW AND AUDIT (Continued)

15.6.5.1 Manager's Supervisory Staff (Continued)

- f. Review violations of Technical Specifications, such reviews to include reports, evaluations and recommendations.
- g. Perform special reviews, investigations or prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.
- h. Review the Facility Fire Protection Program at least once per 24 months to ensure the program meets established commitments and requirements.
- i. Investigate, review, and report on all reportable events.
- j. Review every release of radioactive material to the environment resulting in doses in excess of the annual dose objectives specified in 10 CFR 50, Appendix I or resulting in the special report specified by Section 15.7.8.4.D. Such review will include a summary of evaluation, recommendation and disposition of corrective action to prevent recurrence.
- k. Review all changes to the manuals described in Section 15.7.8.3.

15.6.5.1.9 The Manager's Supervisory Staff shall have the following responsibility:

- a. Serve as an advisory committee to the Manager.
- b. Make recommendations to the Manager for proposals under items a. through d. above. In the event of disagreement between a majority of the Supervisory Staff and decisions by the Manager, the course of action will be determined by the manager and the disagreement recorded in the Staff minutes.

15.7 RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATIONS (RETS)

RETS do not directly expand the responsibilities of the licensed operators of the Point Beach Nuclear Plant Units 1 and 2, and the material contained in this section of these Technical Specifications will not be the subject of SRO/RO licensing examinations.

15.7.1 DEFINITIONS

The definitions for frequently used terminology in these RETS are stated below. These definitions are supplemental to those definitions provided in Section 15.1.

A. Members of the Public

Members of the public include all persons who are not occupationally associated with the plant. This category does not include employees of the utility, its contractors, or vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with the plant.

B. Offsite Dose Calculation Manual (ODCM)

The Offsite Dose Calculation Manual contains the methodology for the determination of gaseous and liquid effluent monitoring alarm or trip setpoints, the methodology for determining compliance with release limits, and the methodology used in the calculation of offsite doses due to radioactive gaseous and liquid effluents.

C. Radioactive Waste Handling

1. Process Control Program (PCP)

The Process Control Program contains the methodologies used to ensure that the processing and packaging of solid radioactive waste will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71, and all other Federal and State regulations governing the disposal of the radioactive waste.

2. Solidification

The conversion of liquid wastes into a form that meets shipping and burial ground requirements.

D. Radiological Effluent and Materials Control and Accountability Program

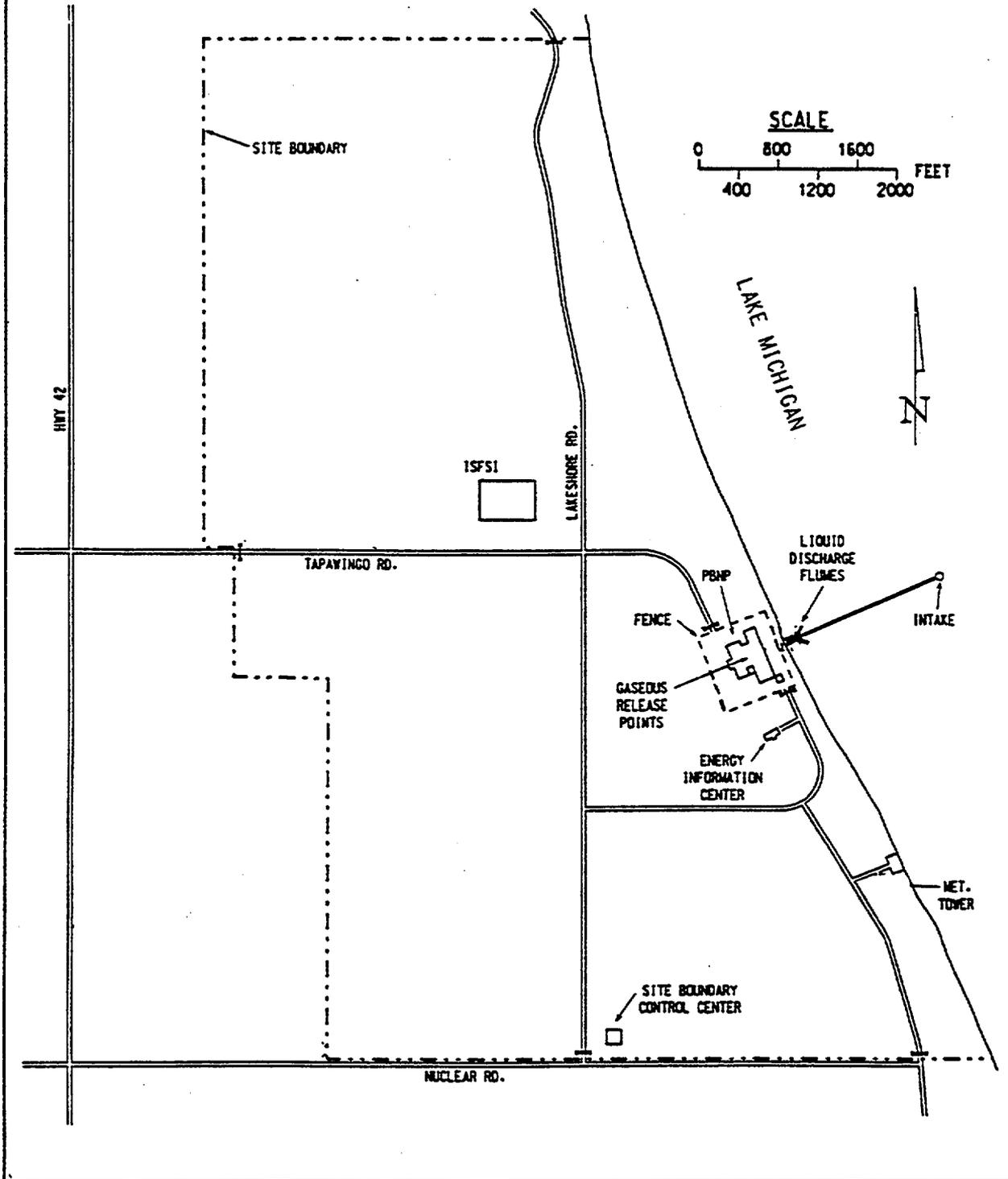
The Radiological Effluent and Materials Control and Accountability Program (REMCAP) specified by 15.7.8.3 shall ensure that all radioactive effluent (materials released via liquid and atmospheric pathways as well as solid materials) from the Point Beach Nuclear Plant are controlled and accounted for in a manner which complies with the applicable regulations and shall ensure that control and accountability for atmospheric and liquid releases are supplemented by environmental sampling and dose calculations. The four major components of the REMCAP are the ODCM, the PCP, the Radiological Environmental Monitoring Program (REMP), and the Radiological Effluent Control Program (RECP). Other supporting guidance, procedures, manuals, or programs may be included or referenced but may not be subject to the program controls specified in 15.7.8.6 and 15.7.8.7.

15.7.2 SITE DESCRIPTION

Figure 15.7.2-1 is a site map for the Point Beach Nuclear Plant Units 1 and 2. The site map shows the site boundary and points within the site boundary from which gaseous and liquid effluents are released. Fence locations are approximate.

15.7.2-1

SITE MAP POINT BEACH NUCLEAR PLANT



Unit 1 - Amendment No. 87, 184
Unit 2 - Amendment No. 107, 188

Figure 15.7.2-1

15.7.3

RADIOACTIVE EFFLUENT MONITORING INSTRUMENTATION
OPERABILITY REQUIREMENTS

Operability requirements have been removed from the Technical Specifications and moved to the REMCAP Manual.

15.7.3-1

Unit 1 - Amendment No. 97, 122, 162, 184
Unit 2 - Amendment No. 101, 125, 166, 188

15.7.4

RADIOACTIVE EFFLUENT MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

Surveillance requirements have been removed from Technical Specifications and moved to the REMCAP Manual.

15.7.5 RADIOACTIVE EFFLUENT RELEASE AND EXPLOSIVE GAS
CONCENTRATION LIMITS

Applicability

Applies to the explosive gas concentration in the radioactive gas decay tanks. Radioactive effluent release limits have been removed from Technical Specifications and placed in the REMCAP Manual.

Objective

To ensure explosive gas concentrations do not exceed the limits of Specification 15.7.5.A.

Specifications

A. Explosive Gas Mixture

The concentration of oxygen in the on-service gas decay tank shall be limited to less than or equal to 4% by volume.

1. If the concentration of oxygen in the on-service gas decay tank is greater than 4% by volume, immediately suspend all additions of waste gases to the on-service gas decay tank.
2. Reduce the oxygen concentration to less than 4% oxygen by volume as soon as possible. If the on-service gas decay tank is at or near capacity and the tank must be isolated to permit the required decay time to conform with release limits, it will not be possible to immediately reduce the oxygen concentration. In this case, the tank will be isolated and the oxygen concentration reduced as soon as the gas decay requirements are satisfied.

15.7.6 RADIOACTIVE EFFLUENT SAMPLING AND ANALYSIS REQUIREMENTS

Sampling and analysis requirements removed from Technical Specifications and placed in the REMCAP Manual.

15.7.7 OPERATIONAL ENVIRONMENTAL MONITORING PROGRAM

The description of the environmental monitoring program has been removed from Technical Specifications and placed in the REMCAP Manual.

15.7.8

ADMINISTRATIVE CONTROLS

15.7.8.1 Duties of the Manager's Supervisory Staff

The duties of the Manager's Supervisory Staff with respect to these radiological effluent technical specifications are listed in Specification 15.6.5.1.8 at Items j. and k.

15.7.8.2 Audits

A. An audit of the activities encompassed by the Radioactive Effluent and Materials Control and Accountability Program (REMCAP), [the Offsite Dose Calculation Manual (ODCM), the Radiological Effluent Control Program (RECP), the Radiological Environmental Monitoring Program (REMP), and the Process Control Program (PCP)] and its implementing procedures shall be performed utilizing either offsite licensee personnel or a consulting firm.

B. The results of the audit above shall be transmitted to the Chief Nuclear Officer and the Chairman of the Offsite Review Committee.

15.7.8.3 Plant Operating Procedures and Programs

The Radioactive Effluent and Materials Control and Accountability Program (REMCAP) shall be established, implemented, and maintained in accordance with the provisions of Technical Specification 15.6.8. REMCAP shall assure that radioactive effluent and waste material from PBNP complies with applicable Federal, State, and burial ground regulations while keeping all exposures to members of the public as low as reasonably achievable (ALARA). This program shall conform to and implement the requirements of PBNP GDC 70 and of 10 CFR 50.34a and 10 CFR 50.36a for the control of radioactive effluents while maintaining doses from these effluents ALARA, shall implement the requirements of 10 CFR 50.34a and General Design Criterion 60 of Appendix A to 10 CFR 50 to suitably control the release and the processing of waste materials, shall conform to the guidance of Appendix I to 10 CFR 50 and PBNP GDC 17 for the assessment of radioactivity in the environs of PBNP, and shall include remedial actions to be taken whenever the program limits are exceeded. REMCAP shall be implemented and maintained under the procedures and methodologies specified in the four (4) manuals/programs listed below and supported, as required, by other procedures. Effluent and environmental monitoring shall be addressed in the Quality Assurance Program.

- a. **Environmental Manual (EM)** - In order to assure conformance with PBNP GDC 17 and with the guidance of Appendix I to 10 CFR 50, this manual shall contain the Radiological Environmental Monitoring Program (REMP) methodology, parameters, and administrative functions required for monitoring, sampling, analyzing, and reporting of radiation levels and radionuclide concentrations in the environment around PBNP. The REMP shall require an annual milk survey and shall require that radioanalyses be conducted by a laboratory participating in an Interlaboratory Comparison Program.

- b. **Radiological Effluent Control Program Manual (RECM)** - The RECM shall contain the Radiological Effluent Control Program (RECP) parameters and methodologies needed in order to assure conformance with 10 CFR 50.36a for the control of effluents and for maintaining doses to the public ALARA and for compliance with 10 CFR 20.1302. The following RECP items are contained in the RECM:
 - 1) radioactive effluent monitoring instrumentation operability, with actions to be taken if operability requirements are not met, and surveillance requirements,
 - 2) radioactive effluent sampling and analyses requirements;
 - 3) limitations on the concentrations of radioactive materials in liquid effluents to the unrestricted area to 10 times the concentration value in Appendix B, Table 2, Column 2 to 10 CFR 20.1001-20.2402;
 - 4) determination of the cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with ODCM methodology and parameters at least every 31 days;
 - 5) assurance that the appropriate portions of the liquid and gaseous effluent treatment system are used whenever the projected doses in a period of 31 days would exceed 2 percent of the 10 CFR 50, Appendix I dose guidelines;
 - 6) limitations to the release rate of radioactive effluent to the atmosphere in order to assure that the following dose rates would not be exceeded at or beyond the site boundary:
 - a. 500 mrem/yr., total body, from noble gases,
 - b. 3000 mrem/yr., skin, from noble gases,
 - c. 1500 mrem/yr., any organ, from I-131, I-133, H-3, and all radioactive particulates with a half-life greater than 8 days;
 - 7) limitations on the annual and quarterly doses to a member of the public at or beyond the site boundary from I-131, I-133, H-3, and radioactive particulates with a half-life > 8

days in effluents released to the atmosphere which conform to 10 CFR 50, Appendix I, dose guidelines, and

- 8) administrative functions and reporting requirements.
- c. Offsite Dose Calculation Manual (ODCM) - The ODCM shall specify that the annual doses from PBNP effluent shall conform to the limits of Appendix I to 10 CFR 50 and of 40 CFR 190, shall contain the parameters and methodology used to calculate the doses to members of the public from PBNP liquid and atmospheric releases in order to demonstrate compliance with these dose limits, and shall contain the methodology to calculate setpoints to ensure that the effluent radionuclide concentrations in unrestricted areas do not exceed the limits specified in the RECP.
- d. Process Control Program (PCP) - The PCP shall provide the methodologies for assuring that the processing and packaging of solid radioactive wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61 and 71, and all other applicable Federal and State regulations governing the disposal of the radioactive waste.

RETS Reporting Requirements

The following written reports shall be submitted to the Administrator, U.S. Nuclear Regulatory Commission Region III with a copy to the Director, Office of Inspection and Enforcement, USNRC, Washington, DC 20555 within the time periods specified.

A. Annual Monitoring Report

A report covering the operation of PBNP for the previous twelve (12) month period, or fraction thereof, ending on December 31, shall be submitted by May 1 of each year containing:

1. RECP

Information relative to the quantities of radioactive liquid and gaseous effluents and solid radioactive waste released from PBNP during the reporting period. The information shall be consistent with the objectives outlined in the ODCM and RECM as well as Section IV.B.1 of Appendix I to 10 CFR 50 and 10 CFR 50.36a.

2. New and Spent Fuel Receipts and Shipments

- a. Number and type of new fuel assemblies received during the reporting period, if any.
- b. Number of spent fuel assemblies shipped off site during the reporting period, if any.

3. REMP

Summaries and results from the REMP for the reporting period. The information shall be consistent with the objectives outlined in the EM and in Sections IV.B.2, IV. B.3, and IV.C of Appendix I to 10 CFR 50.

4. Leak Testing of Sealed Sources

Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.

5. Meteorological Data

Meteorological data shall be kept in file on site for review by the NRC upon request. The data available will include wind speed, wind direction, and atmospheric stability. The data will be in the form of strip charts or hour-by-hour averages stored in electronic form for each of the parameters.

6. REMCAP Changes

A description of changes to the REMCAP, ODCM, EM, RECM or PCP which were implemented and became effective during the reporting period shall be submitted pursuant to Specification 15.7.8.7.

7. Special Circumstances Reports

- a. In accordance with note 7 to RECM Table 3-2, if the Waste Gas Holdup System Explosive Gas Monitor is out of service for greater than 14 days.
- b. In accordance with the EM, factors which render the LLDs unachievable.
- c. In accordance with the EM, failure of the analytical laboratory to participate in an Interlaboratory Comparison Program.

B. Measured Radioactivity Above Notification Levels

If the confirmed level of radioactivity remains above the notification levels specified in the EM, a written report describing the circumstance shall be prepared and submitted within thirty days of the confirmation that a notification level was exceeded.

C. Radioactive Liquid Effluent Treatment

If the radioactive liquid or gaseous effluent treatment system is inoperable and liquid or gaseous effluents are being discharged for 31 days without the treatment required to meet the release limits specified in the RECM, a special report shall be prepared and submitted to the Commission within thirty days which includes the following information:

1. Identification of the inoperable equipment or subsystem and the reason for inoperability.
2. Actions taken to restore the inoperable equipment to operable status.
3. Summary description of actions taken to prevent a recurrence.

D. Radioactive Effluent Releases

If the quantity of radioactive material actually released in liquid or gaseous effluents during any calendar quarter exceeds twice the quarterly limit as specified in the RECM, a special report shall be prepared and submitted to the Commission within thirty days of determination of the release quantity.

15.7.8.5 Major Change to Radioactive Liquid, Gaseous and Solid Waste Treatment Systems

Licensee initiated major changes to the radioactive waste treatment systems (liquid, gaseous, and solid) shall be reported to the U.S. Nuclear Regulatory Commission with the annual update to the FSAR for the period in which the major change was complete. The discussion of each change shall include:

- A. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR Part 50.59;
- B. Information necessary to support the reason for the change;
- C. A description of the equipment, components and processes involved and the interfaces with other plant systems;
- D. An evaluation of the change, which shows how the predicted releases of radioactive materials in liquid effluents and gaseous effluents and/or quantity of solid waste will differ from those previously predicted in the license application and amendments thereto;
- E. An evaluation of the change, which shows the expected maximum exposures to an individual in the unrestricted area and to the general population that differ from those previously estimated in the license application and amendments thereto;
- F. An estimate of the exposure to plant operating personnel as a result of the change.

15.7.8.6 Record Retention

Records of reviews performed for changes made to the REMCAP Manual and to the following REMCAP components; the EM, RECM, ODCM, and PCP; shall be kept for the duration of the operating licenses of Units 1 and 2 of the Point Beach Nuclear Plant.

15.7.8.7 Revisions

A. **Process Control Program**

Revisions to the PCP shall be documented and records of reviews performed for the revision shall be retained as required by 15.7.8.6. The documentation shall contain:

- 1. Information sufficient to support the change together with the appropriate analyses or evaluations justifying the change(s), and
- 2. A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

3. Revisions shall become effective after review and acceptance by the Manager's Supervisory Staff and approval by the Manager - PBNP.

B. Revisions to the EM, ODCM, RECM and REMCAP

1. Revisions to the EM, ODCM, RECM and REMCAP shall be documented and reviews performed of the revision shall be retained as required in 15.7.8.6. The documentation shall contain:
 - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the revision, and
 - b. A determination that the change will maintain the levels of radioactive effluent control required pursuant to 10 CFR 20.1302, 10 CFR 50.36a, Appendix I to 10 CFR 50, and 40 CFR 190.
2. Shall become effective after review and acceptance by the Manager's Supervisory Staff and the approval of the Manager - PBNP,
3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire manual either as part of, or concurrent with, the Annual Monitoring Report for the period of the report in which the revision was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed. Each copy shall indicate the date (e.g., month/year) the revision was implemented.
4. In addition to items 1-3 above, all changes regarding explosive gas must be made via the 50.59 process.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 184

TO FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated April 24, 1996, as supplemented on December 15, 1997, and June 22, 1998, the Wisconsin Electric (WE) Power Company (the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License No. DPR-24 for Point Beach Nuclear Plant, Unit 1, and Facility Operating License No. DPR-27 for Point Beach Nuclear Plant, Unit 2. The proposed amendments would incorporate programmatic controls for radiological effluent and radiological environmental monitoring in the Administrative Controls section of the TS consistent with the requirements of 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50. At the same time, the licensee proposed to transfer the procedural details of TS Section 15.7, "Radiological Effluent Technical Specifications (RETS)." The proposed amendments are based on two new programs, the Radiological Effluent Control Program (RECP) and the Radiological Effluent and Materials Control and Accountability Program (REMCAP). The "Radiological Effluent Control Program Manual" (RECM) will contain the RECP. The REMCAP will link the RECM, the Offsite Dose Calculation Manual (ODCM), the Process Control Program (PCP), and the Radiological Environmental Monitoring Program (REMP). With these changes, the TS related to RETS reporting requirements were simplified. Revised definitions of the ODCM and PCP were proposed consistent with these changes. Additionally, WE proposed relocation of TS 15.7.5 explosive gas monitoring requirements from TS 15.7.5 to the RECP.

The December 15, 1997, and June 22, 1998, submittals provided additional clarifying information and updated TS pages. This information was within the scope of the original *Federal Register* notice and did not change the staff's initial no significant hazards considerations determination.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The Commission's regulatory

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requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

In addition, Section 50.36 contains four criteria to determine which of the design conditions and associated surveillances should be located in the TS as limiting conditions for operation:

1. installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary;
2. a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier;
3. a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; and
4. a structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

As a result, existing TS requirements that fall within or satisfy any of the criteria in 10 CFR 50.36 must be retained in the TS, while those TS requirements that do not fall within or satisfy these criteria may be relocated to other, licensee-controlled documents.

3.0 EVALUATION

WE proposes to revise the radiological effluent TS to include the programmatic controls and administrative requirements and to relocate the details to program manuals that are referenced in the proposed TS in accordance with Generic Letter (GL) 89-01, "Implementation of Programmatic Controls for Radiological Effluent Technical Specifications in the Administrative Controls Section of the Technical Specifications and the Relocation of Procedural Details of RETS to the Offsite Dose Calculation Manual or to the Process Control Program," issued by the NRC on January 31, 1989. Additionally, WE proposes to relocate the explosive gas monitors to a plant program manual referenced in the TS and to include in the proposed TS the requirement to control all changes regarding explosive gas monitoring via the 50.59 process. These changes are in accordance with GL 95-10, "Relocation of Selected Technical Specifications Requirements Related to Instrumentation," issued December 15, 1995. GL 95-10 addresses relocation of the explosive gas monitoring requirements from TS 15.7.5 to other controlled documents such as the proposed RECM.

3.1 RETS

WE proposes to incorporate programmatic controls for radioactive effluent and radiological environmental monitoring in TS 15.7.8 as noted in the guidance provided in GL 89-01. The programmatic controls ensure that programs are established, implemented, and maintained to ensure that operating procedures are provided to control radioactive effluents consistent with the requirements of 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50.

The licensee has confirmed that the detailed procedural requirements addressing limiting conditions for operation, their applicability, remedial actions, associated surveillance requirements, or reporting requirements for the following specifications have been prepared to implement the relocation of these procedural details to the RECM or REMCAP. After relocation, the licensee proposes to require all changes to the REMCAP, RECM, ODCM, Environmental Manual, and PCP be documented, reviewed by the Manager's Supervisory Staff, and approved by the Manager, PBNP, as stated in TS 15.7.8, "Administrative Controls." The required changes have been prepared in accordance with the new Administrative Controls in the TS so that they will be implemented when these amendments are implemented.

WE's proposed changes to the TS are consistent with the guidance provided in GL 89-01 and are addressed below.

The procedural details that have been removed from the TS are not required by the Commission's regulations to be included in TS since the effluent monitors being removed are not the primary means of detecting degradation of the reactor coolant pressure boundary, do not provide an initial condition of a design-basis accident or transient analysis, are not part of the primary success path which is required to operate to mitigate a design-basis accident, and are not contributors to risk. The procedural details have been prepared for incorporation in the RECP and the REMCAP.

TS 15.7.1, "Definitions." Several definitions moved to REMCAP, REMCAP definition added to TS, and "equivalent curie" definition deleted.

TS 15.7.3, "Radioactive Effluent Monitoring Instrumentation Operability Requirements" moved to RECM.

TS 15.7.4, "Radioactive Effluent Monitoring Instrumentation Surveillance Requirements" moved to RECM.

TS 15.7.5, "Radioactive Effluent Release Limits" moved to RECM. References to "equivalent curies" and "Tritium Adjustment" will be deleted because they are no longer used. Instead, dose calculations will be performed.

TS 15.7.6, "Radioactive Effluent Sampling and Analysis Requirements" moved to REMCAP Manual.

TS 15.7.7, "Operational Environmental Monitoring Program" moved to REMCAP.

TS 15.7.8, "Administrative Controls" reporting requirements will be moved to the RECM.

The following specifications that include programmatic requirements are retained and revised in the TS. This is in accordance with the guidance of GL 89-01.

TS 15.6.5, "Review and Audit"

TS 15.6.5.1, "Manager's Supervisory Staff," Item j, will be changed to provide the new location of the release limits. In addition, the review of effluents exceeding annual release limits will be added to this item.

TS 15.6.5.1, Item k

TS 15.7.8, "Administrative Controls." Revisions will be made to this TS; additional material will be added related to the REMCAP.

Additionally, WE proposed some editorial and administrative changes to support the GL 89-01 changes.

Table of Contents

TS 15.3.9, "Effluent Release" revised to show new location of requirements.

Table TS 15.4.1-1, "Minimum Frequencies for Checks, Calibrations, and Tests of Instrument Channels," Footnote 7. This footnote will be changed to reflect the new location for the items removed from TS 15.7.4.

TS 15.4.10, "Operational Environmental Monitoring" revised to reference REMCAP. The reference to "effluent sampling" will be eliminated.

TS 15.7.1, "Operational Environmental Monitoring." The definition of "equivalent curie" will be eliminated because it will no longer be used.

TS 15.7.2, "Site Description." A typographical error will be corrected in the text, and the map will be updated.

The staff finds that the changes included in the proposed TS amendments are consistent with the guidance provided in GL 89-01. Because the control of radioactive effluents continues to be limited in accordance with operating procedures that must satisfy the regulatory requirements of 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50, the staff concludes that the procedural details are not required to be included in the TS by 10 CFR 50.36, these proposed amendments are administrative in nature, and there is no adverse impact on plant safety as a consequence. Accordingly, the staff finds the proposed changes acceptable.

3.2 Relocation of Explosive Gas Monitors TS

3.2.1 Background

The staff evaluation for relocation of the explosive gas monitor TS is included in GL 95-10. This generic letter addresses the relocation of selected TS requirements related to instrumentation. As a result of applying the 10 CFR 50.36 criteria, the NRC determined that several specifications did not warrant inclusion in TS. The staff also concluded that the instrumentation addressed by these specifications is not related to dominant contributors to plant risk.

3.2.2 Relocation of Explosive Gas Monitoring Instrumentation

The proposed relocation of the explosive gas monitoring instrumentation is in accordance with GL 95-10. Since the requirements related to explosive gas monitoring instrumentation do not conform to the 10 FR 50.36 criteria for inclusion in the TS, GL 95-10 recommended that the requirements be relocated to licensee-controlled documents as long as any proposed changes to the explosive gas monitoring instrumentation are controlled in accordance with 10 CFR 50.59.

The relocation of most of the instrumentation related to radioactive gaseous effluent monitoring was addressed in GL 89-01. Relocation of the requirements for explosive gas monitoring instrumentation was not addressed in the guidance provided by GL 89-01. Staff positions regarding the monitoring of explosive gases within the radioactive waste management systems are outlined in Standard Review Plan Section 11.3 and Branch Technical Position ETSB-11-5, "Postulated Radioactive Releases Due to a Waste Gas System Leak or Failure."

The actions required by existing TS typically require alternate sampling, limited operation of the gaseous waste system, and submittal of a special report if the explosive gas monitoring instrumentation does not conform to the limiting condition for operation. The explosive gas monitoring instrumentation requirements address detection of possible precursors to the failure of a waste gas system but do not prevent or mitigate design-basis accidents or transients that assume a failure of or present a challenge to a fission product barrier. Acceptable concentrations of explosive gases are actually controlled by other limiting conditions for operation (e.g., Gaseous Effluents, Explosive Gas Mixture) or by programs described in the "Administrative Controls" section of TS. The requirements related to explosive gas monitoring instrumentation do not conform to the 10 CFR 50.36 criteria for inclusion in the TS. Therefore, licensees may propose to relocate the explosive gas monitoring instrumentation to the UFSAR. WE has proposed to keep explosive gas concentration limits in TS 15.7.5, incorporate the explosive gas monitoring instrumentation requirements into the RECM and added to TS 15.7.8 that the control of changes to explosive gas requirements will be made in accordance with 10 CFR 50.59.

WE proposed relocating the following TS requirements:

TS 15.7.3, "Radioactive Effluent Monitoring Instrumentation Operability Requirements." All material pertaining to explosive gases will be moved to the RECM as listed in the revised TS.

TS 15.7.4, "Radioactive Effluent Monitoring Instrumentation Surveillance Requirements." All material pertaining to explosive gases will be moved to the RECM as listed in the revised TS.

TS 15.7.5, "Radioactive Effluent Release Limits" moved to RECM. The title of this TS will be changed and the explosive gas concentration limits will be copied to the RECM as listed in the revised TS.

TS 15.7.8.7.B.4. will require that any changes related to explosive gases will be made subject to a 10 CFR 50.59 evaluation.

The requirements related to the explosive gas monitoring instrumentation do not meet the criteria for inclusion in the TS under 10 CFR 50.36. These proposed changes are consistent with the guidance provided in GL 95-01 and are acceptable to the staff.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 28620). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). The amendments also change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the

Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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