December 12, 1994

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Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

SUBJECT: AMENDMENT NOS. 158 AND 162 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 - POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2 (TAC NOS. M89702 AND M89703)

Dear Mr. Link:

The Commission has issued the enclosed Amendment Nos. 158 and 162 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2. The amendments revise the Technical Specifications in response to your application dated March 29, 1994.

The amendments modify Point Beach Nuclear Plant Technical Specification (TS) 15.3.2, "Chemical and Volume Control System," by eliminating the necessity for high concentration boric acid and removing the operability requirements for the associated heat tracing. The basis for Section 15.3.2 and applicable surveillances in Table 15.4.1-2 are also revised to support the above changes.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original signed by Allen G. Hansen
Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-266
and 50-301

Enclosures: 1. Amendment No. 158 to DPR-24
2. Amendment No. 162 to DPR-27
3. Safety Evaluation
cc w/encls: See next page

DOCUMENT NAME: G:\ptbeach\PTB89702.AMD
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 12, 1994

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

SUBJECT: AMENDMENT NOS. 158 AND 162 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 - POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2 (TAC NOS. M89702 AND M89703)

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and 50-301

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cc w/encls: See next page

Mr. Robert E. Link, Vice President
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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Two Rivers, Wisconsin 54241

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 158
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   B. Technical Specifications

      The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 158 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: December 12, 1994

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 162
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

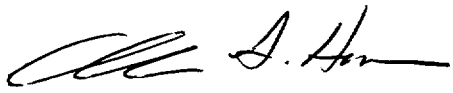
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 162 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: December 12, 1994

ATTACHMENT TO LICENSE AMENDMENT NOS. 158 AND 162

TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
TS 15.3.2-1 through TS 15.3.2-5 (5 pages)	TS 15.3.2-1 through TS 15.3.2-4 (4 pages)
- - - -	Table TS 15.3.2-1 (1 page)
Table TS 15.4.1-2 page 1 through page 4 (4 pages)	Table TS 15.4.1-2 page 1 through page 4 (4 pages)

15.3.2 CHEMICAL AND VOLUME CONTROL SYSTEM

Applicability

Applies to operational status of the chemical and volume control system.

Objective

To define those conditions of the chemical and volume control system necessary to insure safe reactor operation.

Specification

A. When fuel is in the reactor, there shall be at least one flow path to the core for boric acid injection.

B. A reactor shall not be taken critical unless the following chemical and volume control system conditions are met:

1. A minimum of two charging pumps for that reactor shall be operable.
2. System piping and valves shall be operable to the extent of establishing two flow paths from the boric acid tank(s) and/or the refueling water storage tank to the reactor coolant system.
3. If the boric acid tank(s) is required to comply with Specification B.2 above:
   a. One boric acid transfer pump shall be operable per flow path from the boric acid tank(s) with one of the BAST flow paths lined up to supply boric acid to the applicable reactor, and
   b. The boric acid concentration, minimum volume, and solution temperature shall satisfy the requirements of Table 15.3.2-1.

C. A second reactor shall not be taken critical with one reactor already critical unless the following chemical and volume control system conditions are met:

1. A minimum of two charging pumps for that reactor shall be operable.
2. System piping and valves shall be operable to the extent of establishing two flow paths from the boric acid tank(s) and/or the refueling water storage tank(s) to each reactor coolant system.
3. If the boric acid tank(s) is required to comply with Specification C.2 above:
   a. One boric acid transfer pump shall be operable per flow path from the boric acid tank(s) with one of the BAST flow paths per unit lined up to supply boric acid to the applicable reactor, and
   b. The boric acid concentration, minimum volume and solution temperature shall satisfy the requirements of Table 15.3.2-1.

D. During power operation, the requirements of 15.3.2-B and C may be modified to allow the following components to be inoperable for a specified time. If the system is not restored to meet the requirements of 15.3.2-B or C within the time period specified, the appropriate reactor(s), except as otherwise noted, shall be placed in the hot shutdown condition within 6 hours and borated to a shutdown margin equivalent of at least 1.0% delta k/k at cold shutdown, no xenon conditions. If the requirements of 15.3.2-B or C are not satisfied within an additional 7 days, the appropriate reactor(s) shall be placed in the cold shutdown condition within the next 30 hours.

1. One of the two operable charging pumps associated with an operating reactor may be removed from service provided a charging pump associated with that reactor is restored to operable status within 72 hours.
2. One of the two boron injection flow paths specified in B.2 or C.2 may be out of service provided two boron injection flow paths are restored to operable status within 72 hours.
3. One of the boric acid transfer pumps designated in B.3 or C.3 may be out of service provided a boric acid transfer pump is restored to operable status within 72 hours.

Basis

The chemical and volume control system provides control of the reactor coolant system boron inventory. This is normally accomplished by using one or more charging pumps in series with one of the two boric acid transfer pumps. Above cold shutdown conditions, a minimum of two boron injection flow paths are required per unit to insure functional capability in the event that an assumed single active failure renders one of the flow paths inoperable. The boration volume available through any flow path is sufficient to provide the required shutdown margin at cold shutdown, xenon-free conditions from any expected operating condition. The maximum volume requirement is associated with boration from just critical, hot zero or full power, peak xenon with control rods at the insertion limit, to xenon-free, cold shutdown with the highest worth control rod assembly fully withdrawn. This requires approximately 24,100 gallons of 2000 ppm borated water from the refueling water storage tank (RWST) or the concentrations and volumes of borated water specified in Table 15.3.2-1 from the boric acid storage tanks (BASTs).

Available flow paths from the borated water sources to the charging pumps include, but are not limited to, the following:

1. BASTs via one boric acid transfer pump through the normal makeup flow path to the suction of the charging pumps.
2. BASTs via one boric acid transfer pump through the emergency boration flow path to the suction of the charging pumps.
3. RWST via gravity feed through the motor-operated valve to the suction of the charging pumps.
4. RWST via gravity feed through the manual valve to the suction of the charging pumps.

Available flow paths from the charging pumps to the reactor coolant system include, but are not limited to, the following:

1) Charging flow path to the RCS Loop A cold leg.

2) Charging flow path to the RCS Loop B cold leg.

3) Seal injection flow path to the reactor coolant pumps.

Boration of the RCS may also be accomplished via the two boric acid flow paths from the RWST and BASTs through the safety injection (SI) system. Use of these flow paths requires the RCS to be initially depressurized below the SI pump shutoff head.

The amount of boric acid injection must be sufficient to compensate for the addition of positive reactivity from the decay of xenon after a reactor trip from full power in order to maintain the required shutdown margin. This can be accomplished through the operation of one charging pump taking suction from the RWST. Also, the time required for boric acid injection allows for the local alignment of manual valves to provide the necessary flow paths.

The quantity of boric acid specified in Table 15.3.2-1 for each concentration is the quantity necessary per reactor relying on the BAST(s) as a borated water source to borate the reactor coolant to the required cold shutdown concentration at any time in core life. The volume requirements listed in Table 15.3.2-1 are based on the lower concentration in each range. The temperature limits specified

in Table 15.3.2-1 are required to maintain solution solubility at the upper concentration in each range. Heat tracing may be used to maintain solution temperature at or above the limits in Table 15.3.2-1. If the solution temperature of either the flow path or the BAST is not maintained at or above the minimum temperature specified, the affected flow path must be declared inoperable and the appropriate actions in Specification 15.3.2.D followed.

Table 15.3.2-1

Boric Acid Storage Tank(s)

Minimum Volume/Temperature/Concentration

Boric Acid Soln Concentration (Wt%)	Minimum Combined Volume (Gal.)[1]	Minimum Temperature (°F)
3.00 to <3.50	6860[2]	56.0
3.50 to <4.00	5870[2]	62.5
4.00 to <4.50	5120[2]	69.5
4.50 to <5.50	4550[3]	85.0
5.50 to <6.50	3700[3]	97.0
6.50 to <7.50	3150[3]	107.0
7.50 to <8.50	2720[3]	116.0
8.50 to <9.50	2390	123.5
9.50 to <10.50	2140	131.0
10.50 to <11.50	1930	138.0
11.50 to ≤12.50	1750	145.0

(1) Per unit relying on BAST(s) as source of borated water.
(2) Requires more than one BAST per unit.
(3) Requires more than one BAST for two units combined.

TABLE 15.4.1-2

MINIMUM FREQUENCIES FOR EQUIPMENT AND SAMPLING TESTS

		Test	Frequency
1.	Reactor Coolant Samples	Gross Beta-gamma activity (excluding tritium)	5/week[7]
		Tritium activity	Monthly
		Radiochemical $\bar{E}$ Determination	Semiannually [2][10]
		Isotopic Analysis for Dose Equivalent I-131 Concentration	Every two weeks[1]
		Isotopic Analysis for Iodine including I-131, I-133, and I-135	a.) Once per 4 hours whenever the specific activity exceeds 1.0μCi/gram Dose Equivalent I-131 or 100/$\bar{E}$ μCi/gram.[6] b.) One sample between 2 and 6 hours following a thermal power change exceeding 15% of rated power in a one-hour period.
		Chloride Concentration	5/week[8]
		Diss. Oxygen Conc.	5/week[6]
		Fluoride Conc.	Weekly
2.	Reactor Coolant Boron	Boron Concentration	Twice/week
3.	Refueling Water Storage Tank Water Sample	Boron Concentration	Weekly[6]
4.	Boric Acid Tanks	Boron Concentration	Twice/week and after each BAST concentration change when they are being relied upon as a source of borated water.
5.	Spray Additive Tank	NaOH Concentration	Monthly
6.	Accumulator	Boron Concentration	Monthly

Unit 1 - Amendment No. 43, 71, 106, 129, 137, 158
Unit 2 - Amendment No. 33, 76, 109, 133, 161, 162

TABLE 15.4.1-2 (Continued)

		Test	Frequency
7.	Spent Fuel Pit	a) Boron Concentration b) Water Level Verification	Monthly Weekly
8.	Secondary Coolant	Gross Beta-gamma Activity or gamma isotopic analysis	Weekly[6]
		Iodine concentration	Weekly when gross Beta-gamma activity equals or exceeds 1.2 μCi/cc [6]
9.	Control Rods	a) Rod drop times of all full length rods [3]	Each refueling or after maintenance that could affect proper functioning [4]
		b) Rodworth measurement	Following each refueling shutdown prior to commencing power operation
10.	Control Rod	Partial movement of all rods	Every 2 weeks [18]
11.	Pressurizer Safety Valves	Set point	Every five years [11]
12.	Main Steam Safety Valves	Set Point	Every five years [11]
13.	Containment Isolation Trip	Functioning	Each refueling shutdown
14.	Refueling System Interlocks	Functioning	Each refueling shutdown
15.	Service Water System	Functioning	Each refueling shutdown
16.	Primary System Leakage	Evaluate	Monthly [6]
17.	Diesel Fuel Supply	Fuel inventory	Daily
18.	Turbine Stop and Governor Valves	Functioning	Annually [6]
19.	Low Pressure Turbine Rotor Inspection [5]	Visual and magnetic particle or liquid penetrant	Every five years
20.	Boric Acid System	Storage Tank and piping temperatures ≥ temperature required by Table 15.3.2-1	Daily[19]

Unit 1 - Amendment No. 64, 65, 71, 83, 100, 106, 129, 155, 157, 158
Unit 2 - Amendment No. 69, 70, 76, 87, 103, 109, 133, 159, 161, 162

TABLE 15.4.1-2 (Continued)

		Test	Frequency	
21.	PORV Block Valves	a. Complete Valve Cycle b. Open position check	Quarterly [13] Every 72 hours [14]	|
22.	Integrity of Post Accident Recovery Systems Outside Containment	Evaluate	Each refueling cycle	|
23.	Containment Purge Supply and Exhaust Isolation Valves	Verify valves are	Monthly [9] locked closed	|
24.	Reactor Trip Breakers	a. Verify independent operability of automatic shunt and undervoltage trip functions.	Monthly [9]	|
		b. Verify independent operability of manual trip to shunt and undervoltage trip functions.	Each refueling shutdown	
25.	Reactor Trip Bypass Breakers	a. Verify operability of the undervoltage trip function.	Prior to breaker use	|
		b. Verify operability of the shunt trip functions.	Each refueling shutdown	
		c. Verify operability of the manual trip to undervoltage trip functions.	Each refueling shutdown	
26.	120 VAC Vital Instr. Bus Power	Verify Energized[12]	Shiftly	|
27.	Power Operated Relief Valves (PORVs), PORV Solenoid Air Control Valves, and Air System Check	Operate[16]	Each shutdown[15]	|
28.	Atmospheric Steam Dumps	Complete valve cycle	Quarterly	|
29.	Crossover Steam Dump System	Verify operability of each steam dump valve.	Quarterly	|

Unit 1 - Amendment No. 64, 65, 77, 83, 100, 106, 129, 148, 155, 157, 158
Unit 2 - Amendment No. 69, 70, 76, 87, 103, 109, 133, 152, 159, 161, 162

TABLE 15.4.1-2 (Continued)

30.	Pressurizer Heater	Verify that 100KW of heaters are available.	Quarterly
31.	CVCS Charging Pumps	Verify operability of pumps. (17)	Quarterly
32.	Potential Dilution in Progress Alarm	Verify operability of alarm.	Prior to placing plant in cold shutdown.

(1) Required only during periods of power operation.
(2) $\bar{E}$ determination will be started when the gross activity analysis of a filtered sample indicates $\geq 10\mu Ci/cc$ and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than $10\mu Ci/cc$.
(3) Drop test shall be conducted at rated reactor coolant flow. Rods shall be dropped under both cold and hot condition, but cold drop tests need not be timed.
(4) Drop tests will be conducted in the hot condition for rods on which maintenance was performed.
(5) As accessible without disassembly of rotor.
(6) Not required during periods of refueling shutdown.
(7) At least once per week during periods of refueling shutdown.
(8) At least three times per week (with maximum time of 72 hours between samples) during periods of refueling shutdown.
(9) Not required during periods of cold or refueling shutdown, but must be performed prior to exceeding 200°F if it has not been performed during the previous surveillance period.
(10) Sample to be taken after a minimum of 2 EFPD and 20 days power operation since the reactor was last subcritical for 48 hours or longer.
(11) An approximately equal number of valves shall be tested each refueling outage such that all valves will be tested within a five year period. If any valve fails its tests, an additional number of valves equal to the number originally tested shall be tested. If any of the additional tested valves fail, all remaining valves shall be tested.
(12) The specified buses shall be determined energized in the required manner at least once per shift by verifying correct static transfer switch alignment and indicated voltage on the buses.
(13) Not required if the block valve is shut to isolate a PORV that is inoperable for reasons other than excessive seat leakage.
(14) Only applicable when the overpressure mitigation system is in service.
(15) Required to be performed only if conditions will be established, as defined in Specification 15.3.15, where the PORVs are used for low temperature overpressure protection. The test must be performed prior to establishing these conditions.
(16) Test valve operation in accordance with the inservice test requirements of the ASME Boiler and Pressure Vessel Code, Section XI.
(17) Operability of charging pumps is verified by ensuring that the pumps develop the required flowrate, as specified by the In-Service Test program.
(18) Not required to be performed if the reactor is subcritical.
(19) Required only when the BAST(s) are relied upon as a source of borated water.

Unit 1 - Amendment No. 106,129,148,155,157,158
Unit 2 - Amendment No. 109,133,152,159,161,162

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 158 AND 162 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated March 29, 1994, Wisconsin Electric Power Company, the licensee for Point Beach Nuclear Plant (PBNP), submitted a proposal to change the Technical Specifications (TS). Specifically, the proposed changes modify TS 15.3.2, "Chemical and Volume Control System," by decreasing the boric acid storage tank (BAST) boric acid concentration, removing the boric acid system heat tracing and extending the allowed outage time for one of the two charging pumps, one of the two boron injection flow paths, or one of the two boric acid transfer pumps from 24 to 72 hours. Proposed changes to the associated basis for TS 15.3.2 and the applicable surveillances in Table 15.4.1-2 are also included in the submittal.

2.0 EVALUATION

The PBNP concentrated boric acid system consists of three 5000 gallon BASTs and four boric acid transfer pumps (two per unit), along with the associated piping, valves, and heat tracing circuitry. One BAST is normally aligned to each unit and the third BAST can be used as a swing tank and aligned to either unit. Boric acid, currently with a concentration between 11.5 and 12.5 weight percent (wt%), can be injected into the reactor coolant system with the charging pumps through either the emergency, manual, or boric acid blender flow paths, or with the safety injection (SI) pumps. One 275,000 gallon refueling water storage tank (RWST) per unit is also available, as a source of 2000 ppm boric acid.

Upon an SI actuation, the engineered safety features (ESF) logic opens the valve between the BAST and the suction of the SI pumps, aligning the BAST as the initial suction source of the SI fluid. When the BAST low-low level setpoint is reached, the suction line-up from the BAST to SI isolates and the suction valves between the RWST and the SI pumps open automatically.

The proposed system changes include lowering the boric acid concentration of the BAST from the current 11.5-12.5 wt% to the values delineated in the new

Table 15.3.2-1, removing the heat tracing system, and eliminating the logic which automatically opens the valves in the flow path from the BASTs to the SI pumps on an ESF signal.

The proposed TS 15.3.2.B.2 and C.2 ensure that two flow paths and associated sources of borated water are available to maintain long term subcriticality. If the BASTs are used as the source for boration, there has to be sufficient boron to meet the range of concentrations specified in Table 15.3.2-1. The licensee has determined, through the use of the BORDER (Boron Design Requirements) methodology, that 300 pounds of stored boron is sufficient for PBNP's annual fuel cycle.

The minimum temperature requirement listed in Table 15.3.2-1 provides protection against boron precipitation. The temperatures specified in the proposed table represent the solubility temperature plus 5 °F (for boric acid concentrations less than 5 wt%), and the solubility temperature plus 7.8 °F (for boric acid concentrations greater than or equal to 5 wt%). The 7.8 °F margin corresponds to the present margin for 12.5 wt% boric acid solution. The 5 °F solubility temperature margin was chosen by the licensee, because of operational flexibility and the solubility temperature being nearly equal to the ambient temperature.

The licensee determined that these changes could have an effect on the analysis of the loss of coolant accident (LOCA) and steamline break (SLB) events in the Final Safety Analysis Report (FSAR). The licensee re-analyzed these two events, accounting for the proposed changes, and determined that the results remain within the acceptance criteria of the accident analyses.

The licensee also completed an in-house calculation to verify that, for a typical fuel cycle and assuming worse-case conditions, the reactor can be maintained hot subcritical following a reactor trip. Specifically, the licensee determined that the amount of negative reactivity that can be inserted by one charging pump, borating at a minimum speed and using the RWST as its suction source, is greater than the positive reactivity added from the decay of xenon in the longterm.

The applicable surveillances in Table 15.4.1-2 are proposed to be changed by removal of the boric acid heat tracing operability requirements, and by addition of a surveillance to be done, after each boric acid concentration change, during the period that the BASTs are relied upon, as a source of borated water.

The licensee proposed an increase in the allowable outage time in TS 15.3.2.D for one of the two boration flow paths from 24 to 72 hours. The licensee has based this change on the design capability of the boration system to provide sufficient volume through one flow path, meeting the required shutdown margin, xenon-free, from any operating condition. This change is consistent with NUREG-0452, Revision 4, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors" (STS).

The licensee also proposed an increase in the TS 15.3.2.D hot shutdown requirement time from 3 hours to 6 hours, an additional requirement for

boration, as part of the hot shutdown requirement, an increase in the time to restore operability from 48 hours to 7 days, and a decrease in the cold shutdown requirement from 48 hours to 30 hours. These changes are consistent with the current staff positions for the proposed system configurations, and with the STS.

The licensee has demonstrated that these changes do not adversely affect the FSAR accident analyses and that, with the changes, the plant will still be able to mitigate the consequences of accidents and anticipated operational occurrences. Based on this demonstration by the licensee, the staff finds these changes acceptable. In addition, the staff agrees with the licensee that the proposed changes to the bases are consistent with, and support the above changes.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (59 FR 37091). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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