

December 21, 1994

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Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

SUBJECT: AMENDMENT NOS. 159 AND 163 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 - POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2 (TAC NOS. M90539 AND M90540)

Dear Mr. Link:

The Commission has issued the enclosed Amendment Nos. 159 and 163 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated September 12, 1994.

The amendments modify Point Beach Nuclear Plant Technical Specification (TS) 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," by incorporating allowed outage times similar to those contained in NUREG-1431, Revision 0, "Westinghouse Owner's Group Improved Standard Technical Specifications," and by clarifying the operability requirements for the service water pumps. The changes also clarify the completion times for placing a unit in hot or cold shutdown, if a limiting condition for operation cannot be met.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original signed by Allen G. Hansen
Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-266
and 50-301

- Enclosures: 1. Amendment No. 159 to DPR-24
2. Amendment No. 163 to DPR-27
3. Safety Evaluation

cc w/encls: See next page

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Mr. Robert E. Link, Vice President
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

Ernest L. Blake, Jr.
Shaw, Pittman, Potts & Trowbridge
2300 N Street, N.W.
Washington, DC 20037

Mr. Gregory J. Maxfield, Manager
Point Beach Nuclear Plant
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, Wisconsin 54241

Town Chairman
Town of Two Creeks
Route 3
Two Rivers, Wisconsin 54241

Chairman
Public Service Commission
of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702

Regional Administrator
U.S. NRC, Region III
801 Warrenville Road
Lisle, Illinois 60532-4531

Resident Inspector's Office
U.S. Nuclear Regulatory Commission
6612 Nuclear Road
Two Rivers, Wisconsin 54241



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 159
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 12, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 159, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: December 21, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 12, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 45 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: December 21, 1994

ATTACHMENT TO LICENSE AMENDMENT NOS. 159 AND 163
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

TS 15.3.3-2 through
TS 15.3.3-4 (5 pages)

TS 15.3.3-6

TS 15.3.3-8 through
TS 15.3.3-10 (3 pages)

INSERT

TS 15.3.3-2 through
TS 15.3.3-4 (4 pages)

TS 15.3.3-6

TS 15.3.3-8 through
TS 15.3.3-10 (3 pages)

- f. The isolation valves in the discharge header of the high head safety injection system are in the open position.
 - g. All valves, interlocks, and piping associated with the above components and required to function during accident conditions are operable.
 - h. During conditions of operation with reactor coolant system pressure in excess of 1,000 psig, the source of AC power shall be removed from the accumulator isolation valves MOV-841A and B at the motor control center and the valves shall be open.
 - i. Power may be restored to MOV-841A and B for the purpose of valve testing or maintenance providing the testing and maintenance is completed and power is removed within four hours.
2. During power operation, the requirements of 15.3.3.A.1, Items b and c, may be modified to allow one of each of the following components to be inoperable at any one time. If the system is not restored to meet the requirements of 15.3.3.A.1 within the time period specified, the reactor shall be placed in the hot shutdown condition within six hours and in cold shutdown within 36 hours.
- a. One accumulator may be isolated to perform a check valve leakage test or be otherwise inoperable for a period of up to one hour. Before isolating an accumulator, the other accumulator isolation valve shall be checked open.
 - b. One safety injection pump may be out of service, provided the pump is restored to operable status within 72 hours. The other safety injection pump shall be operable.
 - c. Any valve in these systems required to function during accident conditions may be inoperable provided repairs are completed within 72 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable.
3. During power operation, the requirements of 15.3.3.A.1, Items d and e, may be modified to allow one of each of the following components to be inoperable at any one time. If the component is not restored to meet

the requirements of 15.3.3.A.1 within the time specified, the reactor shall be placed in the hot shutdown condition within six hours. The reactor shall be maintained in a condition with reactor coolant temperatures greater than 350°F, unless one residual heat removal loop is being relied upon to provide redundancy for decay heat removal. In this case the reactor shall be maintained between 350°F and 140°F.

- a. One residual heat removal pump may be out of service, provided the pump is restored to operable status within 72 hours. The other residual heat removal pump shall be operable.
- b. One residual heat exchanger may be out of service for a period of no more than 72 hours.
- c. Any valve in the system, required to function during accident conditions, may be inoperable provided repairs are completed within 72 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable.

B. Containment Cooling and Iodine Removal Systems

1. A reactor shall not be made critical, except for low temperature physics tests, unless the following conditions associated with that reactor are met:
 - a. The spray additive tank contains not less than 2675 gal. of solution with a sodium hydroxide concentration of not less than 30% by weight.
 - b. Two containment spray pumps are operable.
 - c. Four accident fan-cooler units are operable.
 - d. All valves and piping, associated with the above components and required to function during accident conditions, are operable.
2. During power operation, the requirements of 15.3.3.B-1 may be modified to allow any one of the following components to be inoperable at any one time. If the system is not restored to meet the requirements of 15.3.3.B-1 within the time period specified, the reactor shall be placed in the hot shutdown condition within six hours and in cold shutdown within 36 hours.
 - a. One accident fan cooler may be out of service provided that cooler is returned to operable status within 48 hours. The other accident fan coolers shall be operable before initiating maintenance on the inoperable accident fan cooler.
 - b. One containment spray pump may be out of service provided the pump is restored to operable status within 72 hours. The remaining containment spray pump shall be operable before initiating maintenance on the inoperable pump.

- c. Any valve required for the functioning of the system during accident conditions may be inoperable provided repairs are completed within 72 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable.

C. Component Cooling System

Single Unit Operation

1. One reactor shall not be made critical unless the following conditions are met:
 - a. The two component cooling pumps assigned to that unit are operable.
 - b. Either the component cooling heat exchanger associated with the unit together with one of the shared spare heat exchangers are operable or the two shared spare heat exchangers are operable.
 - c. All valves, interlocks and piping associated with the above components, and required for the functioning of the system during accident condition, are operable.
2. During power operation, the requirements of 15.3.3.C-1 may be modified to allow one of each of the following conditions at any one time. If the system is not restored to meet the conditions of 15.3.3.C-1 within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 15.3.3.C-1 are not

- a. Four service water pumps are operable, two from each train.
 - b. All necessary valves, interlocks and piping required for the functioning of the Service Water System during accident conditions are also operable.
2. During power operation, the requirements of 15.3.3.D-1 may be modified to allow one of the following components to be inoperable at any one time. If the system is not restored to meet the conditions of 15.3.3.D-1 within the time period specified, both reactors will be placed in the hot shutdown condition within six hours and in cold shutdown within 36 hours.
- a. One of the four required service water pumps may be out of service provided a pump is restored to operable status within 24 hours.
 - b. One of the two loop headers may be out of service for a period of 24 hours.
 - c. A valve or other passive component may be out of service provided repairs can be completed within 48 hours.

Basis

The normal procedure for starting the reactor is, first, to heat the reactor coolant to near operating temperature, by running the reactor coolant pumps. The reactor is then made critical by withdrawing control rods and/or diluting boron in the coolant.⁽¹⁾ With this mode of start-up, the energy stored in the reactor coolant during the approach to criticality is substantially equal to that during power operation and therefore to be conservative most engineered safety system components and auxiliary cooling systems, shall be fully operable. During low temperature physics tests there is a negligible amount of stored energy in the reactor coolant, therefore an accident comparable in severity to the Design Basis Accident is not possible, and the engineered safety systems are not required.

Assuming the reactor has been operating at full rated power for at least 100 days, the magnitude of the decay heat decreases as follows after initiating hot shutdown.*

<u>Time After Shutdown</u>	<u>Decay Heat % of Rated Power</u>
1 min.	3.6
30 min.	1.55
1 hour	1.25
8 hours	0.7
48 hours	0.4

*Based on ANS 5.1-1979, "Decay Heat Power in Light-Water Reactors"

Thus, the requirement for core cooling in case of a postulated loss-of-coolant accident while in the hot shutdown condition is significantly reduced below the requirements for a postulated loss-of-coolant accident during power operation. Putting the reactor in the hot shutdown condition significantly reduces the potential consequences of a loss-of-coolant accident, and also allows more free access to some of the engineered safety system components in order to effect repairs.

When the failures involve the residual heat removal system, in order to insure redundant means of decay heat removal, the reactor system may remain in a condition with reactor coolant temperatures between 500 and 350°F so that the reactor coolant loops and associated steam generators may be utilized for redundant decay heat removal. However, when the remaining RHR loop must be relied upon for redundant decay heat removal capability, reactor coolant temperatures shall be maintained between 350°F and 140°F.

With respect to the core cooling function, there is some functional redundancy for certain ranges of break sizes.⁽²⁾

The operability of the Refueling Water Storage Tank (RWST) as part of the ECCS ensures that a sufficient supply of borated water is available for injection by the ECCS in the event of either a LOCA or a steamline break. The limits on RWST

minimum volume and boron concentration ensure that: (1) sufficient water is available within containment to permit recirculation cooling flow to the core; (2) the reactor will remain subcritical in the cold condition (68 to 212 degrees-F) following a small break LOCA assuming complete mixing of the RWST, RCS, spray additive tank, containment spray system piping and ECCS water volumes with all control rods inserted except the most reactive control rod assembly (ARI-1); (3) the reactor will remain subcritical in the cold condition following a large break LOCA (break flow area greater than 3 ft²) assuming complete mixing of the RWST, RCS, ECCS water and other sources of water that may eventually reside in the sump post-LOCA with all control rods assumed to be out (ARO); and (4) long term subcriticality is maintained following a steamline break assuming ARI-1 and fuel failure is precluded.

The containment cooling function is provided by two independent systems: (a) fan coolers and (b) containment spray which, with sodium hydroxide addition, provides the iodine removal function. During normal power operation, only three of the four fan coolers are required to remove heat lost from equipment and piping within the containment.⁽³⁾ In the event of a Design Basis Accident, any one of the following combinations will provide sufficient cooling to reduce containment pressure: (1) four fan coolers, (2) two containment spray pumps, (3) two fan coolers plus one containment spray pump.⁽⁴⁾ Sodium hydroxide addition via one spray pump reduces airborne iodine activity sufficiently to limit off-site doses to acceptable values. One of the four fan coolers is permitted to be inoperable for up to 48 hours during power operation.

Specification 15.3.3.B.2.c requires valves that provide the duplicate function be operable prior to initiating repairs on an inoperable valve. For the specific case of the containment spray pump discharge (SI-860) valves, SI-860A and SI-860D provide duplicate functions. Valves SI-860B and SI-860C are not required for system operability. Hence, prior to removing valve SI-860A from service, valve SI-860D must be operable and vice versa.

The component cooling system is different from the other systems discussed above in that the components are so located in the Auxiliary Building as to be accessible for repair after a loss-of-coolant accident. One component cooling water pump together with one component cooling heat exchanger can accommodate the heat

removal load on one unit either following a loss-of-coolant accident, or during normal plant shutdown. If during the post-accident phase the component cooling water supply is lost, core and containment cooling could be maintained until repairs were effected.⁽⁵⁾

A total of six service water pumps are installed, only two of which are required to operate during the injection and recirculation phases of a postulated loss-of-coolant accident,⁽⁶⁾ in one unit together with a hot shutdown or normal operation condition in the other unit. For either reactor to be critical, four service water pumps must be operable. Two of the pumps must be powered from the 'A' train, and the other two must be powered from the 'B' train.

References

- (1) FSAR Section 3.2.1
- (2) FSAR Section 6.2
- (3) FSAR Section 6.3.2
- (4) FSAR Section 6.3
- (5) FSAR Section 9.3.2
- (6) FSAR Section 9.6.2



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 159 AND 163 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated September 12, 1994, Wisconsin Electric Power Company, the licensee, proposed to amend the Point Beach Nuclear Plant (PBNP), Unit Nos. 1 and 2, Technical Specifications (TSs). The proposed amendment would modify TS 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," by incorporating allowed outage times similar to those contained in NUREG-1431, Revision 0, "Westinghouse Owner's Group Improved Standard Technical Specifications" (ISTS), and by clarifying the operability requirements for the service water pumps. The proposed changes would also clarify the completion times for placing a unit in hot or cold shutdown, if a limiting condition for operation cannot be met. In addition, the licensee proposed changes to the related bases to ensure consistency and clarity.

2.0 EVALUATION

2.1 TS 15.3.3.A.2

The allowed outage times (AOTs) and the unit shutdown requirements related to accumulators and safety injection (SI) pumps are being modified. The AOTs are being increased from 24 hours to 72 hours, and the shutdown requirements are being changed from 3 hours to 6 hours to reach hot shutdown, and from 48 hours to 36 hours to reach cold shutdown.

The proposed 72-hour AOTs are consistent with the current staff position, represent a reasonable time to repair a component, and are consistent with ISTS. Therefore, the proposed AOTs are acceptable. The proposed 6-hour period to reach hot shutdown is reasonable, based on operating experience, so that plant systems are not challenged, and is consistent with current staff positions and ISTS. Therefore, the hot shutdown requirement is acceptable. The proposed 36-hour period to reach cold shutdown is a reduction in the allowed time stated in the PBNP custom TSs. This change is acceptable because it is conservative, and is bounded by the current staff position.

2.2 TS 15.3.3.A.3

The AOTs and the unit shutdown requirements related to residual heat removal (RHR) pumps, heat exchangers and valves (AOTs only) are being modified. The AOTs are being increased from 24 hours to 72 hours for the pumps and valves, and from 48 hours to 72 hours for the heat exchangers. The requirement to reach hot shutdown is being changed from 3 hours to 6 hours for both pumps and heat exchangers. The requirement to maintain the reactor coolant between 350 °F and 500 °F after an additional 48 hours is being modified to maintain a temperature between 350 °F and 540 °F, immediately upon entering the condition.

The proposed 72-hour AOTs for the RHR components are consistent with the current staff position, represent a reasonable time to repair a component, and are consistent with ISTS. Therefore, the proposed AOTs are acceptable. The proposed 6-hour period to reach hot shutdown is reasonable, based on operating experience, so that plant systems are not challenged, and is consistent with current staff positions and ISTS. Therefore, the hot shutdown requirement is acceptable.

The upper bound of the temperature band for the reactor coolant is being changed from 500 °F to 540 °F. This change brings the temperature range into agreement with the hot shutdown mode definition in the custom PBNP TSs and with the PBNP design bases, correcting an error in the TSs. Therefore, it is acceptable. The change in the time to be in the specified temperature range is acceptable, because it is conservative and consistent with current operating practice at PBNP.

2.3 TS 15.3.3.B.2

The AOTs and the unit shutdown requirements related to containment fan coolers, spray pumps and associated valves are being modified. The AOT is being increased from 48 hours to 72 hours for a spray pump, and from 24 hours to 72 hours for a valve. The shutdown requirements related to the coolers, pumps and valves are being changed from 3 hours to 6 hours to reach hot shutdown, and from 48 to 36 hours to reach cold shutdown. In addition, an exception related to valves out-of-service is being deleted.

The 72-hour completion time takes into account the redundant heat removal capability provided by the containment spray and cooler systems, provides a reasonable time for repairs, and is consistent with the current staff position and with ISTS. Therefore, the completion time is acceptable.

The proposed 6-hour period to reach hot shutdown is reasonable, based on operating experience, so that plant systems are not challenged, and is consistent with current staff positions and ISTS. Therefore, the hot shutdown requirement is acceptable. The proposed 36-hour period to reach cold shutdown is a reduction in the allowed time stated in the PBNP custom TSs. This change is acceptable, because it is conservative and is bounded by the current staff position.

The removal of the valve out-of-service exception is acceptable, because it will no longer apply since the AOTs for the pumps and the valves will be the same.

2.4 TS 15.3.3.D.1

The requirement for four service water (SW) pumps to be operable is being modified to state that two pumps from each train must be operable. This is a conservative change which ensures redundancy and is consistent with ISTS. Therefore, the change is acceptable. In addition, the footnote which was only applicable during the Fall 1994 refueling outage is being removed. As this is an administrative change only, it is acceptable.

2.5 TS 15.3.3.D.2

The unit shutdown requirements related to SW pumps, valves and associated components are being modified. The requirements are being changed from 3 hours to 6 hours to reach hot shutdown, and from 48 to 36 hours to reach cold shutdown.

The proposed shutdown requirements are reasonable, based on operating experience, to reach the required conditions in an orderly manner without challenging plant systems, and are consistent with ISTS. Therefore, they are acceptable.

2.6 Bases for TS 15.3.3

The staff agrees with the licensee, that the proposed changes to the bases are consistent with and support the above changes.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (59 FR 53844). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Hansen

Date: December 21, 1994