

April 16, 1993

Docket Nos. 50-266
and 50-301

DISTRIBUTION

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

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OC/LFMB
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Region III, DRP

Dear Mr. Link:

SUBJECT: AMENDMENT NOS. 138 AND 142 TO FACILITY OPERATING LICENSE NOS. DPR-24
AND DPR-27 (TAC NOS. M76021 AND M76022)

The Commission has issued the enclosed Amendment Nos. 138 and 142 to Facility
Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant,
Unit Nos. 1 and 2. The amendments revise the Technical Specifications in
response to your application dated December 7, 1989.

These amendments eliminate the requirements for testing components in the
opposite train when certain components in a train are out of service. These
components are safety injection pumps, residual heat removal pumps,
containment spray pumps, and containment accident pumps. The change also
eliminates testing redundant valves in safety injection, residual heat
removal, and containment spray systems when a valve is inoperable.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will
be included in the Commission's next biweekly Federal Register notice.

Sincerely,
Original signed by Anthony T. Gody, Jr.

Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 138 to DPR-24
- 2. Amendment No. 142 to DPR-27
- 3. Safety Evaluation

cc w/enclosures:
See next page

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LA: PD3-3 PE: PD3-3 PM: PD3-3 D: PD3-3: DRPW
PKreutzer RSamworth AGody/bj JHannon
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BC: OTS/B CGrimes OGC-OWF
3/25/93 4/16/93
CWSM #93-33

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 16, 1993

Docket Nos. 50-266
and 50-301

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

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AND DPR-27 (TAC NOS. M76021 AND M76022)

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These amendments eliminate the requirements for testing components in the opposite train when certain components in a train are out of service. These components are safety injection pumps, residual heat removal pumps, containment spray pumps, and containment accident pumps. The change also eliminates testing redundant valves in safety injection, residual heat removal, and containment spray systems when a valve is inoperable.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Anthony T. Gody, Jr.".

Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 138 to DPR-24
2. Amendment No. 142 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Robert E. Link
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated December 7, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

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P PDR

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 138, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: April 16, 1993



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-27

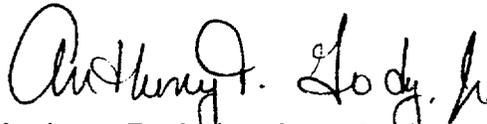
1. The Nuclear Regulatory Commission (the Commission) has found that:
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 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony T. Gody, Jr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: April 16, 1993

ATTACHMENT TO LICENSE AMENDMENT NOS. 138 AND 142
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

15.3.3-2

15.3.3-2

15.3.3-2a

15.3.3-2a

15.3.3-3

15.3.3-3

15.3.3-3a

15.3.3-3a

15.3.3-4

15.3.3-4

15.3.3-7

15.3.3-7

- f. The isolation valves in the discharge header of the high head safety injection system are in the open position.
 - g. All valves, interlocks, and piping associated with the above components and required to function during accident conditions are operable.
 - h. During conditions of operation with reactor coolant system pressure in excess of 1,000 psig, the source of AC power shall be removed from the accumulator isolation valves MOV-841A and B at the motor control center and the valves shall be open.
 - i. Power may be restored to MOV-841A and B for the purpose of valve testing or maintenance providing the testing and maintenance is completed and power is removed within four hours.
2. During power operation, the requirements of 15.3.3.A.1, Items b and c, may be modified to allow one of each of the following components to be inoperable at any one time. If the system is not restored to meet the requirements of 15.3.3.A.1 within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 15.3.3.A.1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition.
- a. One accumulator may be isolated for a period of up to one hour to permit a check valve leakage test. Before isolating an accumulator, the other accumulator isolation valve shall be checked open.
 - b. One safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. The other safety injection pump shall be operable.
 - c. Any valve in these systems required to function during accident conditions may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable.
3. During power operation, the requirements of 15.3.3.A.1, Items d and e, may be modified to allow one of each of the following components to be inoperable at any one time. If the component is not restored to meet

the requirements of 15.3.3.A.1 within the time specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 15.3.3.A.1 are not satisfied within an additional 48 hours, the reactor shall be maintained in a condition with reactor coolant temperatures between 500 and 350°F, unless one residual heat removal loop is being relied upon to provide redundancy for decay heat removal. In this case the reactor shall be maintained between 350°F and 140°F.

- a. One residual heat removal pump may be out of service, provided the pump is restored to operable status within 24 hours. The other residual heat removal pump shall be operable.
- b. One residual heat exchanger may be out of service for a period of no more than 48 hours.
- c. Any valve in the system, required to function during accident conditions, may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable.

B. Containment Cooling and Iodine Removal Systems

1. A reactor shall not be made critical, except for low temperature physics tests, unless the following conditions associated with that reactor are met:
 - a. The spray additive tank contains not less than 2675 gal. of solution with a sodium hydroxide concentration of not less than 30% by weight.
 - b. Two containment spray pumps are operable.
 - c. Four accident fan-cooler units are operable.
 - d. All valves and piping, associated with the above components and required to function during accident conditions, are operable.
2. During power operation, the requirements of 15.3.3.B-1 may be modified to allow any one of the following components to be inoperable at any one time. If the system is not restored to meet the requirements of 15.3.3.B-1 within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 15.3.3.B-1 are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition.
 - a. One accident fan cooler may be out of service provided that cooler is returned to operable status within 48 hours. The other accident fan coolers shall be operable before initiating maintenance on the inoperable accident fan cooler.

- b. One containment spray pump may be out of service provided the pump is restored to operable status within 48 hours. The remaining containment spray pump shall be operable before initiating maintenance on the inoperable pump.

- c. Any valve required for the functioning of the system during accident conditions may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable. (Exception: If a spray pump is removed from service per b above, valves associated with that train may be removed from service for the period specified for the pump.)

C. Component Cooling System

Single Unit Operation

1. One reactor shall not be made critical unless the following conditions are met:
 - a. The two component cooling pumps assigned to that unit are operable.
 - b. Either the component cooling heat exchanger associated with the unit together with one of the shared spare heat exchangers are operable or the two shared spare heat exchangers are operable.
 - c. All valves, interlocks and piping associated with the above components, and required for the functioning of the system during accident condition, are operable.
2. During power operation, the requirements of 15.3.3.C-1 may be modified to allow one of each of the following conditions at any one time. If the system is not restored to meet the conditions of 15.3.3.C-1 within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 15.3.3.C-1 are not

The operable status of the various systems and components is to be demonstrated by periodic tests, defined by Specification 15.4.5. A large fraction of these tests will be performed while the reactor is operating in the power range. If a component is found to be inoperable it will be possible in most cases to effect repairs and restore the system to full operability within a relatively short time. For a single component to be inoperable does not negate the ability of the system to perform its function, but it reduces the redundancy provided in the reactor design and thereby limits the ability to tolerate additional equipment failures. If it develops that (a) the inoperable component is not repaired within the specified allowable time period or (b) a second component in the same or related system is found to be inoperable, the reactor will initially be put in the hot shutdown condition to provide for reduction of the decay heat from the fuel, and consequent reduction of cooling requirements after a postulated loss-of-coolant accident. This will also permit improved access for repairs in some cases. After a limited time in hot shutdown, if the malfunction(s) are not corrected, the reactor will be placed in the cold shutdown condition, utilizing normal shutdown and cooldown procedures. In the cold shutdown condition there is no possibility of an accident that would release fission products or damage the fuel elements.

The specified repair times do not apply to regularly scheduled maintenance of the engineered safety systems, which is normally to be performed during refueling shutdowns. The limiting times to repair are based on:

- 1) Assuring with high reliability that the safety system will function properly if required to do so.
- 2) Allowances of sufficient time to effect repairs using safe and proper procedures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 138 AND 142 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated December 7, 1989 Wisconsin Electric Power Company applied for an amendment to the technical Specifications for the Point Beach Nuclear Plant. The amendment would allow removal of one train of certain safety-related equipment from service without testing the redundant equipment to demonstrate operability. The amendment does not authorize any change to the limiting conditions for operation but only affects operability testing of the redundant components. The application identified four specific components. The four components are safety injection (SI) pumps, residual heat removal (RHR) pumps, containment spray (CS) pumps, and containment accident fans. The licensee also proposed deleting the requirement to test redundant valves in the SI, RHR, and CS systems when a valve is inoperable. Specifically, the following changes are proposed:

Change 1.

Technical Specification 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," item A.2.b. specifies that during power operation "One safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. The other safety injection pump shall be tested to demonstrate operability prior to initiating repair of the operable pump." If the pump cannot be returned to operable status within the specified time period, the reactor shall be placed in the hot shutdown condition.

The licensee would change this to require "One safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. The other safety injection pump shall be operable."

Change 2.

Currently item 15.3.3.A.2.c., referring to accumulators and safety injection pumps, specifies that "Any valve in these systems required to function during accident conditions may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs all valves in the system that provide the duplicate function shall be tested to demonstrate operability."

WEPCo would change this second sentence to state: "Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable."

Change 3.

Currently item 15.3.3.A.3.a specifies that "One residual heat removal pump may be out of service, provided the pump is restored to operable status within 24 hours. The other residual heat removal pump shall be tested to demonstrate operability prior to initiating the repair of the inoperable pump."

The licensee proposed that this be changed to require "One residual heat removal pump may be out of service, provided the pump is restored to operable status within 24 hours. The other residual heat removal pump shall be operable."

Change 4.

Currently item 15.3.3.A.3.c., referring to residual heat removal pumps and to residual heat exchangers, specifies that "Any valve in the system, required to function during accident conditions, may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves in the system that provide the duplicate function shall be tested to demonstrate operability."

WEPCo would change this second sentence to state: "Prior to initiating repairs, all valves in the system that provide the duplicate function shall be operable."

Change 5.

Item 15.3.3.B.2.a. specifies: "One accident fan cooler may be out of service provided that cooler is returned to operable status within 48 hours. The accident fan coolers shall be tested to demonstrate operability before initiating maintenance on the inoperable fan cooler and shall be tested once every 24 hours thereafter until all accident fan coolers are in an operable status or the reactor is shutdown."

WEPCo proposed changing the second sentence to specify: "The other accident fan coolers shall be operable before initiating maintenance on the inoperable accident fan cooler."

Change 6.

Item 15.3.3.B.2.b. specifies: "One containment spray pump may be out of service provided the pump is restored to operable status within 48 hours. The remaining containment spray pump shall be tested to demonstrate operability before initiating maintenance on the inoperable pump and shall be tested once every 24 hours thereafter until both pumps are in an operable status or the reactor is shutdown."

WEPCo proposed changing the second sentence to read: "The remaining containment spray pump shall be operable before initiating maintenance on the inoperable pump."

Change 7.

Item 15.3.3.B.2.c., referring to the accident fan coolers and containment spray pumps, specifies: "Any valve required for the functioning of the system during accident conditions may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves in the system that provide duplicate function shall be tested to demonstrate operability...."

WEPCo proposed changing the second sentence to read: "Prior to initiating repairs, all valves shall be operable."

Change 8.

WEPCo proposes revising the corresponding bases section to remove the reference to testing redundant components.

2.0 EVALUATION

In its application, WEPCo pointed out that the technical specifications include testing and surveillance requirements which establish operability.

Staff technical position:

To take a redundant system out of service for testing, if the first system fails, creates the risk of the second system also failing. It has been observed that failures of the second system are often related to the test itself and are not indicative that the redundant system would have failed should it have been needed. Immediate redundant testing can be deleted from technical specifications if routine surveillance frequencies are adequate to ensure operability. Additionally, both systems would be out of service for the duration of the testing of the redundant component.

Acceptance of a change to technical specifications deleting testing of the redundant component is contingent on the licensee's adherence to procedures for administratively verifying operability of items whose redundant counterparts have been declared inoperable.

Thus if the technical specifications contain adequate provision for operability testing and if the licensee has appropriate administrative procedures in place, the elimination of redundant testing will be found acceptable. The standard technical specification (Standard Technical Specifications for Westinghouse Pressurized Water Reactors, NUREG-0452, latest revision) provides guidance in establishing an appropriate level of testing.

Changes 1 and 3:

For the Point Beach Nuclear Plant, the safety injection system constitutes the emergency core cooling system and WEPCo uses the terms interchangeably (FSAR Par. 6.2.1). The principal components of the safety injection system are the two accumulators, the two safety injection (high head) pumps, and the two residual heat removal (RHR) (low head) pumps. Also included are appurtenances such as the RHR heat exchangers, a boric acid storage tank and a refueling water storage tank in addition to piping and valving. Surveillance requirements for the safety injection pumps and the residual heat removal pumps are found in Technical Specification section 15.4.5, "Emergency Core Cooling System and Containment Cooling System Tests." Specification 15.4.5.I.A.1 requires that system tests be performed during major fuel reloadings in accordance with a specified procedure. The specified procedure utilizes a test signal and requires verification that all components have received the safety injection signal in the proper sequence and timing. This is done annually at Point Beach.

Specification 15.4.5.II.A.1 requires that the safety injection pumps and the residual heat removal pumps be started and operated on recirculation flow monthly. During this monthly test, each pump must reach its required developed head at miniflow and operate for at least 15 minutes on the miniflow line.

Testing which the staff found acceptable in the Westinghouse Standard Technical Specification includes a requirement to test at least once per 18 months during shutdown by verifying that the safety injection pump starts automatically upon receipt of a test signal (4.5.2.e). It is also required that each of the safety injection pumps develops the indicated pressure on recirculation flow when tested pursuant to specification 4.0.5 (Inservice Testing).

The standard technical specification also requires testing during shutdown after making any modification to the ECCS system that may alter flow characteristics.

The Point Beach technical specifications permit one safety injection pump to be out of service for 24 hours (page 15.3.3-2, item 2.b.). Then the licensee must proceed to hot shutdown within 3 hours, and to cold shutdown within the next 48 hours, commensurate with the requirements of Technical Specification 15.3.0. In the standard technical specification, one ECCS subsystem may be inoperable for 72 hours.

The testing interval required by the Point Beach technical specifications is no greater than the interval which the staff has accepted in the standard technical specification. The Point Beach allowance for inoperability is more restrictive than the allowance in the standard technical specification. Therefore, the deletion of the redundant testing is acceptable without consideration being given to additional testing.

Changes 2 and 4:

Surveillance requirements in the technical specifications ensure that valves in specified systems are operable. Technical Specification section 15.4.5.I.A. requires that safety injection system tests be performed during reactor shutdown for major refuelings. The test will be considered satisfactory if control board indication and visual observations indicate that all components have received the safety injection signal in the proper sequence and timing. That is, the appropriate pump motor breakers shall have opened and closed, and all valves shall have completed their travel.

Section 15.4.5.II.B. specifies additional component tests. Refueling water storage tank outlet valves are to be checked during each monthly test of pumps and the accumulator check valves are to be checked for operability during each refueling shutdown.

Section 15.4.5.I.B. requires a comparable surveillance for the containment spray system.

Inservice testing also addresses valves.

The standard technical specifications require periodic verification of valve position, in addition to testing by actuation, at least once per 18 months. The surveillances in the Point Beach technical specifications are performed at a frequency comparable to that in the Standard Technical Specifications. Therefore, elimination of the redundant testing does not have a significant impact on safety and is acceptable.

Change 5:

Containment accident fans

Surveillance requirements in the technical specifications ensure that the containment accident fans are operable. The accident fan coolers are called containment fan coolers in the surveillance section of the Point Beach Technical Specifications. Section 15.4.5.I.C. requires that containment fan cooler accident fans shall be tested monthly to verify operability. Acceptable performance shall be that the accident fan starts and running current is verified. Section 15.4.5.I.C. also requires that each fan cooler unit shall be tested at each refueling to verify proper operation of the backfit dampers and the service water bypass valves.

The standard technical specification, Section 4.6.2.3, requires that each group of containment cooling fans shall be demonstrated operable at least once per 31 days by starting each fan group from the control room and verifying that each fan group operates at least 15 minutes. The licensee should also verify that cooling water flow is adequate. The standard technical specification also requires that once per 18 months operability should be demonstrated by verifying that each fan group starts automatically on a test signal.

The standard technical specification requires at least two independent groups of an least two fans to each group. The standard allows one group to be inoperable for 7 days if both containment spray systems are operable. It further allows two groups of fans or one group of fans and one containment spray system to be inoperable for 72 hours.

Since the Point Beach technical specification requires operability testing on a frequency comparable to the standard and since the allowed inoperable period is of shorter duration than the standard, the proposed TS change is acceptable.

The Point Beach technical specification includes monthly testing of the fans. This is an adequate frequency on which to base an operability determination. It is noted that the existing Point Beach technical specifications do not explicitly require that the fans be tested to start on an accident signal. In a phone conversation on March 11, 1993, the licensee stated that during the performance of licensee/operations refueling test (ORT) procedure #3, which satisfies the requirements of Technical Specifications 15.4.5.1.a.1 and 15.4.6.a.2, containment cooler fan breaker operation is verified upon receipt of an SI signal.

Based on the frequency of testing of the fans, and verification of fan breaker operation during ORT procedure #3, the deletion of the redundant testing requirement will not have a significant impact on safety and is, therefore, acceptable.

Change 6:

Containment Spray pumps

Point Beach Technical Specification 15.4.5.I.B.1. requires that the containment spray system tests shall be performed during reactor shutdowns for major fuel reloading. Section 15.4.5.II.A.1. requires that containment spray pumps shall be started and operated on recirculation flow monthly. Performance is acceptable if the pumps start, reach their required head at miniflow, and operate for at least 15 minutes on the miniflow line.

The standard technical specification Section 4.6.2.1 requires that each containment spray system be demonstrated operable by verifying adequate pressure when doing the IST program testing. At least once per 18 months the licensee should verify that each spray starts automatically on a specified test signal.

The standard technical specification allows one containment spray pump to be inoperable for 72 hours. The Point Beach technical specifications allow one containment spray pump to be out of service for 48 hours.

Based on the frequency of testing of the containment spray pumps in the Point Beach Technical Specifications, the deletion of the redundant testing requirement will not have a significant impact on safety and is, therefore, acceptable.

Change 7

The valves required for the functioning of the accident fan coolers and the containment spray pumps are not identified in the technical specifications. The surveillance requirement for the containment spray system specifies that the test of the system during refueling be done with isolation valves in the spray supply lines blocked closed. Subpart 15.4.5.II.B. requires that the refueling water storage tank outlet valves be tested in performing the pump tests. It further requires that the spray additive valves shall be checked for operability monthly. The accumulator check valves will be checked for operability during each refueling shutdown. The concentrated boric acid tank isolation valves to the safety injection system shall be tested monthly.

The standard technical specification requires verification of proper valve position once per 31 days, but requires testing only once per 18 months to verify that each automatic valve in the flow path actuates to its correct position on a specified test signal. The 72-hour allowance for inoperability of the containment spray system would apply to inoperability due to a valve taken out of service.

The Point Beach technical specification surveillance of the containment fan cooler system specifies that the test of each unit during refueling verify proper operation of the backdraft dampers and the service water bypass valves.

The standard technical specification for the containment cooling system does not specifically address surveillance of valves.

Because the technical specifications do address testing of valves, the staff believes that an adequate basis would exist for establishing operability of redundant valves without additional testing. Therefore deletion of the redundant testing requirement will not have a significant impact on safety and is, therefore, acceptable.

Change 8

The change to the basis section eliminates the explanation of why testing of redundant components is included. Based on the analyses of individual changes, the staff finds the change to the basis will have no impact on safety and is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (56 FR 31415). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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