

May 6, 1993

Docket Nos. 50-266  
and 50-301

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Mr. Robert E. Link, Vice President  
Nuclear Power Department  
Wisconsin Electric Power Company  
231 West Michigan Street, Room P379  
Milwaukee, Wisconsin 53201

Dear Mr. Link:

SUBJECT: CORRECTION - AMENDMENT NOS. 137 AND 141 TO FACILITY OPERATING  
LICENSE NOS. DPR-24 AND DPR-27 (TACS M85314 AND M85315)

The Commission issued Amendment Nos. 137 and 141 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2 on March 26, 1993. Inadvertently, two different amendment numbers were cited throughout the amendment for Unit No. 1. The correct number for DPR-24 is Amendment No. 137. The amendment package has been reissued in its entirety.

We are sorry for any inconvenience this error may have caused you.

Sincerely,

Original Signed By:

Anthony T. Gody, Jr., Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 137 to DPR-24
2. Amendment No. 141 to DPR-27
3. Safety Evaluation

cc w/enclosures:

See next page

LA:PDIII-3  
MRushbrook  
4/23/93

PM:PDIII-3  
AGody, Jr.:sw

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Anthony T. Gody, Jr., Project Manager  
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Office of Nuclear Reactor Regulation

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Mr. Robert E. Link  
Wisconsin Electric Power Company

Point Beach Nuclear Plant  
Unit Nos. 1 and 2

cc:

Ernest L. Blake, Jr.  
Shaw, Pittman, Potts & Trowbridge  
2300 N Street, N.W.  
Washington, DC 20037

Mr. Gregory J. Maxfield, Manager  
Point Beach Nuclear Plant  
Wisconsin Electric Power Company  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Town Chairman  
Town of Two Creeks  
Route 3  
Two Rivers, Wisconsin 54241

Chairman  
Public Service Commission  
of Wisconsin  
Hills Farms State Office Building  
Madison, Wisconsin 53702

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
Office of Executive Director  
for Operations  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Resident Inspector's Office  
U.S. Nuclear Regulatory Commission  
6612 Nuclear Road  
Two Rivers, Wisconsin 54241



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 26, 1993

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Wisconsin Electric Power Company  
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The Commission has issued the enclosed Amendment Nos. 137 and 141 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated January 19, 1993.

These amendments revise the degraded voltage relay setpoints in Table 15.3.5-1, revise the actions required if the conditions specified in Table 15.3.5-3, Items 4.a and 4.b are not met, add the bus designations to Tables 15.3.5-1 and 15.3.5-3, and correct typographical errors which were contained in amendments 55 and 60 issued on September 30, 1981.

A temporary waiver of compliance with the conditions in Technical Specification Table 15.3.5-3, "Emergency Cooling," Item 4.a, "Degraded Voltage (4.16 kV)," was requested and granted on January 14, 1993. The NRC confirmed the granting of this waiver in a letter to WEPCo dated January 14, 1993. The waiver allowed the continued operation of the plant with non-conservative degraded voltage relay setpoints for the safety buses. This waiver was based, in part, on your expressed intent to submit the application for the amendment to the degraded voltage setpoint specification. With the issuance of this amendment the temporary waiver of compliance is terminated.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Anthony T. Gody, Jr.".

Anthony T. Gody, Jr., Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

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P PDR



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WASHINGTON, D. C. 20555

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Wisconsin Electric Power Company

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Unit Nos. 1 and 2

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2300 N Street, N.W.  
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6612 Nuclear Road  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated January 19, 1993 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

**B. Technical Specifications**

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 137, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony T. Gody, Jr., Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of issuance: March 26, 1993





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 141  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated January 19, 1993 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

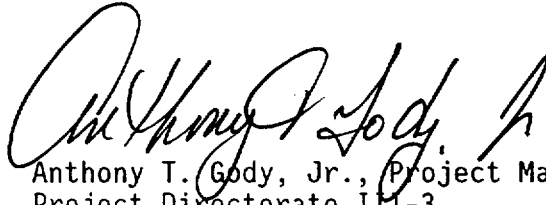
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 141 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Anthony T. Gody, Jr., Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of issuance: March 26, 1993

ATTACHMENT TO LICENSE AMENDMENT NOS. 137 AND 141  
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27  
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

Table TS 15.3.5-1  
Table TS 15.3.5-3

INSERT

Table TS 15.3.5-1 (Pg 2 of 2)  
Table TS 15.3.5-3

Unit 1 - Amendment No. 55, 57, 58, 74, 78, 124, 137  
 Unit 2 - Amendment No. 50, 51, 52, 79, 82, 127, 141

TABLE 15.3.5-1 (Continued)  
 (Page 2 of 2)

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>CHANNEL</u>	<u>SETTING LIMIT</u>
9.	Degraded Voltage (4.16 KV) (A05, A06)	Disconnection of affected bus from offsite power	>3959 volts + 1/2% Time delay: less than 60 sec at 0-100% of voltage setting
10.	Loss of Voltage		
	a. 4.16 KV (A05, A06)	Disconnection of affected bus from offsite power Start Diesel	a. 3220 volts ±2% Time Delay: <1 sec ±10% at 0-100% of voltage setting
	b. 480 V (A05, A06)	Load shedding	b. 256 volts ±3% Time delay: ≤.75 sec ±10% at 0 volts <3.5 sec ±20% at 90% of voltage setting

Unit 1 - Amendment No. 38, 47, 55, 137  
 Unit 2 - Amendment No. 50, 52, 50, 141

TABLE 15.3.5-3

EMERGENCY COOLING

NO.	FUNCTIONAL UNIT	1 NO. OF CHANNELS	2 NO. OF CHANNELS TO TRIP	3 MINIMUM OPERABLE CHANNELS	4 MINIMUM DEGREE OF REDUNDANCY	5 PERMISSIBLE BYPASS CONDITIONS	OPERATOR ACTION IF CONDITIONS OF COLUMN 3 OR 4 CANNOT BE MET
3.	AUXILIARY FEEDWATER (continued)						
d.	Safety Injection Signal Starts Motor Driven Pumps						S.I. INITIATING FUNCTIONS AND REQUIREMENT AS IN 1. ABOVE
4.	SAFETY RELATED ELECTRICAL BUSES						
a.	Degraded Voltage (4.16KV) (A05, A06)	3/bus	2/bus	2/bus	1/bus		****
b.	Loss of Voltage (4.16KV) (A05, A06)	2/bus	1/bus	1/bus	1		****
c.	Loss of Voltage (480V) (B03, B04)	3/bus	2/bus	2/bus	1		Hot Shutdown***

\*\* - Must activate 2 switches simultaneously.

\*\*\* - If minimum conditions are not met within 24 hours after reaching hot shutdown, the unit shall be in cold shutdown within 48 hours of the event causing the unit shutdown.

\*\*\*\* - Declare the associated emergency diesel generator inoperable for the affected bus. The applicable Limiting Condition for Operation (LCO) shall be entered. Separate LCOs may be entered for the Degraded Voltage and Loss of Voltage functions.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 137 AND 141 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

A request for a temporary NRR Waiver of Compliance from the conditions in Technical Specification Table 15.3.5-3, "Emergency Cooling," Item 4.a, "Degraded Voltage (4.16 kV)," was granted to Wisconsin Electric Power Company on January 14, 1993. It allowed the continued operation of the Point Beach Nuclear Plant, Units 1 and 2 with non-conservative degraded voltage relay setpoints for a number of the safety buses. The staff granted the Temporary Waiver of Compliance based on conservative licensee compensatory actions and an understanding that it would not be in the interest of public health and safety to start and operate both emergency diesel generators continuously as the Technical Specifications would require. The NRC confirmed the granting of a temporary waiver of compliance in a letter to WEPCo dated January 14, 1993. This waiver was also based, in part, on the licensee's expressed intent to submit an application for an amendment to the degraded voltage setpoint specification.

By letter dated January 19, 1993, Wisconsin Electric Power Company requested a revision to the Technical Specifications on an exigent basis to change the degraded voltage relay setpoints and the corresponding action statement. Title 10 of the Code of Federal Regulations, Part 50.91, stipulates in section (a)(6) that, where the Commission finds that exigent circumstances exist in that a licensee and the Commission must act quickly and that time does not permit the Commission to publish a Federal Register notice allowing 30 days for public comment, and it also determines that the amendment involves no significant hazards considerations, it will, among others, either issue a Federal Register notice providing an opportunity for hearing and allowing at least two weeks for prior public comment or use the local media to provide reasonable notice to the public in the area surrounding a licensee's facility. The staff concluded, however, that a 30-day Federal Register notice period was appropriate to allow sufficient opportunity for prior public comment on the proposed permanent Technical Specification change request since the licensee was granted relief from Technical Specification 15.3.5-3, Item 4.a in the letter dated January 14, 1993, granting the Temporary Waiver of Compliance removing the actual exigent nature of the situation. Therefore, the staff denied the licensee's request for handling the license amendment in an exigent manner.

Specifically, the proposed amendment would revise Technical Specification Table 15.3.5-1, "Engineered Safety Features Initiation Instrument Setting Limits," Item 9, "Degraded Voltage (4.16 KV)." Item 9 now requires the degraded voltage relay setpoints to be set at greater than or equal to 3875 volts plus or minus 2%. The proposed amendment would change this setpoint to 3959 volts plus or minus 1/2%.

The proposed amendment would also revise Technical Specification Table 15.3.5-3, "Emergency Cooling," Item 4.a, "Degraded Voltage (4.16KV)," and Item 4.b, "Loss of Voltage (4.16 KV)." Item 4.a specifies the number of instrument channels which must be operable for degraded voltage protection for each 4.16 KV safeguards bus and the degree of redundancy which must exist. Item 4.b imposes similar requirements for the loss-of-voltage protection channels. If these conditions cannot be met, the current specifications permit continued operation for up to 7 days provided both emergency diesel generators are otherwise operable and the associated diesel generator is operating and providing power to the affected bus. If the condition is corrected within 7 days, then the affected unit is required to be placed in hot shutdown. The proposed amendment would change the action statement to both items to read: "Declare the associated emergency diesel generator inoperable for the affected bus. The applicable Limiting Condition for Operation (LCO) shall be entered. Separate LCOs may be entered for the Degraded Voltage and Loss of Voltage functions."

In addition, the amendment would add the bus designations in Technical Specification Tables 15.3.5-1, and 15.3.5-3.

## 2.0 EVALUATION

Based on the analysis performed by ABB Impell Calculation 0970-150-007, Revision 0, Wisconsin Electric Power Company has determined that the existing settings for the degraded voltage relays installed on the 4160 V safety-related buses may be too low to provide adequate protection for all safety-related equipment. An undervoltage relay setting of greater than or equal to 3959 V +/- 0.5 % is necessary to protect equipment while still providing an adequate level of equipment reliability. The new, higher setpoint for the degraded voltage relays will allow the removal of offsite power and allow the transfer of the affected safety-related buses to the emergency diesel generators at an appropriate voltage level to protect safety equipment. This value for the degraded voltage relay setting is based on the worst case assumptions for the safety grade bus loading and system configuration and assures the protection of safeguard equipment from degraded voltage conditions.

The licensee states that the field measurements and verification will be performed during 1993 to verify and refine the existing calculation inputs assumptions, and results. These measurements will include quantifying actual relay, calibration, and potential transformer tolerances. Because of the equipment involved, portions of this verification effort will be required to be performed during the 1993 maintenance and refueling outages for both units and is expected to be completed by the end of the Unit 2 refueling outage in

November 1993. Following completion of this verification effort, additional action, if needed, will be taken to insure this relay setting continues to provide the appropriate level of equipment protection as well as the reliability of offsite power to the safety grade buses. The licensee further states that its planned actions and schedule will be provided to the NRC at that time. The staff has evaluated the licensee's submittal and has determined that the licensee has provided justification for the change in the degraded voltage relay setpoints in Technical Specification Table 15.3.5-1 and, therefore, this change is acceptable.

The licensee has also proposed a change to the actions required if the conditions specified in Technical Specification Table 15.3.5-3, Items 4.a and 4.b, are not met. Technical Specification Table 15.3.5-3, Item 4.a, specifies that two of the three existing channels of degraded voltage protection for each 4.16 kV safeguard bus be operable and a minimum degree of redundancy of one channel per bus be maintained. If these conditions cannot be met, the specification permits continued operation for up to 7 days provided both emergency diesel generators are otherwise operable and the associated diesel generator is operating and providing power to the affected bus. If the condition is not corrected within 7 days, then the affected unit is required to be placed in hot shutdown. The same actions are required if the conditions of Item 4.b, "Loss of Voltage (4.16 kV)," are not met. The licensee has determined that operation for an extended period of time with all four of the safety-related 4160 V buses supplied from the emergency diesel generators was not consistent with maintaining the minimum level of plant safety. With the proposed change, the associated emergency diesel generator is administratively declared inoperable for the affected bus only. The LCO is entered with respect to the affected unit only. This is appropriate since the emergency diesel generators are shared between the two units. Inoperable degraded voltage or loss of voltage protection on a bus in one unit does not prevent the emergency diesel generator from performing its intended function with respect to a loss of voltage or degraded voltage condition on the unaffected unit's bus or from responding properly to an actuation of the unaffected protective function on the affected bus. The LCOs for inoperable degraded voltage and inoperable loss of voltage channels may be entered separately. It is unlikely that both protective functions would be inoperable concurrently. The licensee states that channel check, calibration, and test frequencies specified in Table 15.4.1-1, "Minimum Frequencies for Checks, Calibrations and Test of Instrument Channels," ensure that an inoperable channel is promptly detected and returned to service. This LCO, the required actions, and the supporting surveillance are consistent with the W STS and are, therefore, acceptable.

In addition, the licensee has made editorial changes to Technical Specification Table 15.3.5-1, Items 9 and 10, and Technical Specification Table 15.3.5-3, Items 4.a, b, and c. These changes add the bus designations for the 4.16 kV and 480 V buses, as appropriate, to the bus. The staff finds this proposed change to be acceptable.



### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N.K. Trehan

Date: March 26, 1993