UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 14, 1993

Docket Nos. 50-266
and 50-301

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

Dear Mr. Link:

SUBJECT: TECHNICAL SPECIFICATION BASIS CHANGE - CONTROL ROD AND POWER DISTRIBUTION LIMITS (TACS M85198 AND M85199)

By letter dated October 30, 1992, Wisconsin Electric Power Company submitted a proposed change to the Basis for Point Beach Nuclear Plant Technical Specification 15.3.10, "Control Rod and Power Distribution Limits." Specifically, in the subsection on axial power distribution, a change would be made to the description of how the axial flux difference (AFD) is derived from the plant process computer. The current basis states that "the computer determines the one minute average of each of the operable excore detector outputs..." As revised, the basis would explain that "the computer determines the AFD for each of the operable excore channels..." Although not explicit in the revised wording, the new determination uses the current instantaneous value of the axial flux difference. The basis for making the change is that you have made a change to the calculation procedure.

The former calculation method used a one minute average of the axial flux difference for each channel. This method results in a delay of as much as two minutes between the time the axial flux difference actually changes and the time the computer indicates the change. The new calculation method uses current nuclear instrumentation system readings and updates the display once every four seconds. This essentially eliminates the delay in recognizing a change.

We understand that prior to implementation of the change in the software for the plant process computer system, a safety evaluation was performed in accordance with 10 CFR 50.59.

The staff finds the revision to Technical Specification Basis page 15.3.10-12 acceptable on the basis that the details of the process computer software are not subject to NRC review and NRC has not imposed any constraint on the axial flux difference calculation methodology. The revised technical specification basis page is enclosed with this letter.

Sincerely,

ORIGINAL SIGNED BY:

Anthony T. Gody, Jr., Sr. Project Manager
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Enclosure:
As stated

cc w/enclosure:
See next page

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NAME	MRushbrook	AGodyJr:sw	RJones	JHannon	
DATE	6/8/93	6/8/93	6/10/93	6/14/93	/93

OFFICIAL RECORD

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Axial Power Distribution

The limits on axial flux difference (AFD) assure that the axial power distribution is maintained such that the $F_Q(Z)$ upper bound envelope of F_Q^{LIMIT} times the normalized axial peaking factor [K(Z)] is not exceeded during either normal operation or in the event of xenon redistribution following power changes. This ensures that the power distributions assumed in the large and small break LOCA analyses will bound those that occur during plant operation.

Provisions for monitoring the AFD on an automatic basis are derived from the plant process computer through the AFD monitor alarm. The computer determines the AFD for each of the operable excore channels and provides a computer alarm if the AFD for at least 2 of 4 or 2 of 3 operable excore channels are outside the AFD limits and the reactor power is greater than 50 percent of Rated Power.

cc:

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