September 7, 1993

Docket Nos. 50-266
and 50-301

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

Dear Mr. Link:

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SUBJECT: AMENDMENT NOS. 141 AND 145 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 (TACS M76919 AND M76920)

The Commission has issued the enclosed Amendment Nos. 141 and 145 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated March 30, 1990.

These amendments change Section 15.4.5, "Emergency Core Cooling System and Containment Cooling System Tests," to allow testing of the safety injection system while motor breakers are racked in and operable. Section 15.4.6, "Emergency Power System Periodic Tests," is revised to relocate requirements for performing tests of emergency lighting, to eliminate the requirement that emergency lighting be tested during the test of the automatic start of the diesel generators and to delete notations that are no longer applicable as an administrative improvement. Section 15.6.12, "Environmental Qualification," is deleted.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by Allen G. Hansen
Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 141 to DPR-24
2. Amendment No. 145 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page
*See previous concurrence

PD3-3:LA	PD3-3:(A)PM	PD3-3:PM	OTSB	PD3-3:PD	OGC
*MRushbrook	*TMcGinty	*AHansen/bj	CGrimes	JHannon	
7/27/93	7/29/93	7/30/93 9/7/93	8/23/93	8/23/93	8/31/93

DOCUMENT NAME: G:\PTBEACH\PTB76919.AMD #93-134

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 7, 1993

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and 50-301

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
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2. Amendment No. 145 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

cc:

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6612 Nuclear Road
Two Rivers, Wisconsin 54241

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 141
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 141, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: September 7, 1993

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 145
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 145 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Allen G. Hansen, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: September 7, 1993

ATTACHMENT TO LICENSE AMENDMENT NOS. 141 AND 145

TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
15.4.5-1	15.4.5-1
15.4.6-1	15.4.6-1
15.4.6-2	15.4.6-2
15.4.6-3	15.4.6-3
15.4.6-4	15.4.6-4
15.6.12-1	-----

15.4.5 EMERGENCY CORE COOLING SYSTEM AND CONTAINMENT COOLING SYSTEM TESTS

Applicability:

Applies to testing of the Emergency Core Cooling System and the Containment Cooling System.

Objective:

To verify that the subject systems will respond promptly and perform their design functions, if required.

Specification:

I. System Tests

A. Safety Injection System

1. System tests shall be performed during reactor shutdowns for major fuel reloading. The test shall be performed in accordance with the following procedure:

a. With the Reactor Coolant System pressure less than or equal to 350 psig and temperature less than or equal to 350°F, a test safety injection signal will be applied to initiate operation of the system. The motor breakers for the safety injection and residual heat removal pumps may be placed in the "test" position or racked in and operable for this test.

2. The test will be considered satisfactory if control board indication and visual observations indicate that all components have received the safety injection signal in the proper sequence and timing.

15.4.6 EMERGENCY POWER SYSTEM PERIODIC TESTS

Applicability

Applies to periodic testing and surveillance requirements of the emergency power system.

Objective

To verify that the emergency power system will respond promptly and properly when required.

Specification

The following tests and surveillance shall be performed as stated:

A. Diesel Generators

1. Manually-initiated start of the diesel generator, followed by manual synchronization with other power sources and assumption of load by the diesel generator shall not exceed 2850KW. This test will be conducted monthly with a minimum running time of 30 minutes on each diesel generator. Normal plant operation will not be affected.

2. Automatic start of each diesel generator, load shedding, and restoration to operation of particular vital equipment, initiated by an actual interruption of normal AC station service power supplies to associated engineered safety systems busses together with a simulated safety injection signal. In addition, after the diesel generator has carried its load for a minimum of 5 minutes, automatic load shedding and restoration of vital loads are tested again by manually tripping the diesel generator output breaker. This test will be conducted during reactor shutdown for major fuel reloading of each reactor to assure that the diesel generator will start and assume required load in accordance with the timing sequence listed in FSAR Section 8.2 after the initial starting signal.

3. The proper operation of Emergency Lighting, including the automatic transfer switch for DC lights, will be demonstrated during each reactor shutdown for a major fuel reloading.

4. Each diesel generator shall be given an inspection, at least annually, following the manufacturer's recommendations for this class of stand-by service.

5. Each fuel oil transfer pump shall be run monthly.

The above tests will be considered satisfactory if all applicable equipment operates as designed.

B. Safety-Related Station Batteries

These surveillance specifications are applicable to all four safety-related station batteries: D05, D06, D105, and D106; and the safety-related station swing battery D305.

1. Every month the voltage of each cell (to the nearest 0.05 volt), the specific gravity and temperature of a pilot cell in each battery and each battery voltage shall be measured and recorded.

2. Every 3 months the specific gravity, the height of electrolyte, and the amount of water added, for each cell, and the temperature of every fifth cell, shall be measured and recorded.

3. At each time data is recorded, new data shall be compared with old to detect signs of abuse or deterioration.

4. Each Safety-Related Station Battery shall be demonstrated OPERABLE:

   a. At least once per 18 months (SERVICE TEST) by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual or simulated emergency loads for the design duty cycle.

b. At least once per 60 months (PERFORMANCE TEST) by verifying that the battery capacity is at least 80% of the manufacturer's rating. This performance discharge test may be performed in lieu of the battery service test.

c. Annual performance discharge tests of battery capacity shall be given to any battery that shows signs of degradation or has reached 85% of the service life expected for the application. Degradation is indicated when the battery capacity drops more than 10% of rated capacity form its previous performance test, or is below 90% of the manufacturer's rating. When performance tests are required, they may be performed in lieu of the battery service test.

Basis

The tests specified are designed to demonstrate that the diesel generators will provide power for operation of equipment. They also assure that the emergency generator system controls and the control systems for the safeguards equipment will function automatically in the event of a loss of all normal AC station service power.

The testing frequency specified will be often enough to identify and correct any mechanical or electrical deficiency before it can result in a system failure. The fuel supply and starting circuits and controls are continuously monitored and any faults are alarm indicated. An abnormal condition in these systems would be signaled without having to place the diesel generators themselves on test.

Station batteries will deteriorate with time, but precipitous failure is extremely unlikely. The surveillance specified is that which has been demonstrated over the years to provide an indication of a cell becoming unserviceable long before it fails. If a battery cell has deteriorated or if a connection is loose, the voltage under load will drop excessively indicating replacement or maintenance.

A Service Test, performed at least every 18 months, demonstrates adequate battery capacity to supply power to loads required during the most demanding duty cycle. This design duty cycle occurs upon an actuation of safeguards loads in one unit coincident with a loss of off-site power. The design duty cycle is defined further in FSAR, Section 8.2.

A Performance Test will also be conducted at least every 60 months. The Performance Test is a constant discharge rate capacity test which allows comparison with the manufacturer's rating of the battery. This test is the best indicator of the effects of aging on battery capacity. Provisions are made in these specifications to change the test periodicity to annual when the battery is degraded or when the battery reaches that point in its service life at which capacity degradation with time is accelerated. Operability is satisfactorily demonstrated by achieving a capacity of at least 80% of the manufacturer's rating. Since the Performance Test entirely bounds the battery loads applied during a Service Test. when a Performance Test is conducted, the Service Test for that battery's current test cycle may be omitted.

These surveillance specifications are applicable to all five of the safety-related station batteries: D05, D06, D105, D106, and the swing battery D305.

DC emergency lights are provided in certain safeguards equipment areas which must be attended to during a loss of all AC power. The emergency lighting test verifies that the automatic transfer switch operates properly and provides DC power to the DC emergency lights.

Reference
FSAR, Section 8.2

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 141 AND 145 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated March 30, 1990, Wisconsin Electric Power Company (WEPCo) requested an amendment to Facility Operating License Nos. DPR-24 and DPR-27 to revise the Technical Specifications (TS) for the Point Beach Nuclear Plant (PBNP). The proposed amendments would make three unrelated sets of changes.

Change 1: Section 15.4.5, "Emergency Core Cooling System and Containment Cooling System Tests," would be revised by changing the system testing requirement for the safety injection system to allow the test to be performed while the motor breakers for the safety injection and the residual heat removal pumps are racked in and operable. The current TS reads:

1. System tests shall be performed during reactor shutdowns for major fuel reloading. The test shall be performed in accordance with the following procedure:

a. With the Reactor Coolant System Pressure less than or equal to 350 psig and temperature less than or equal to 350 °F, a test safety injection signal will be applied to initiate operation of the system. The motor breakers for the safety injection and residual heat removal pumps are placed in the "test" position for this test.

The proposed amendments would change the final sentence above to read: "The motor breakers for the safety injection and residual heat removal pumps may be placed in the 'test' position or racked in and operable for this test." Placing breakers in the "test" position prevents actual starting of the pumps. As amended, pumps would be allowed to start.

Change 2: Section 15.4.6, "Emergency Power System Periodic Tests," would be revised by relocating from subsection A.2 to a new subsection, A.3, the requirement that during each reactor refueling a checkout of emergency lighting be performed. Items currently numbered 3 and 4 would be renumbered 4 and 5. The change also eliminates the requirement that the test of the emergency lighting be done during the test of the automatic start of the diesel generators. The amendments would change the term "changeover relay" to read "automatic transfer switch." A description of the emergency lighting

checkout procedures would be added to the corresponding bases section of the technical specifications.

Change 3: Section 15.6.12, "Environmental Qualification," would be deleted. Currently this section reads as follows:

A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of Licenses DPR-24 and DPR-27 dated October 24, 1980.

B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

Administrative Improvement: Additionally, as an administrative improvement to the Technical Specifications, notations relevant to the surveillance interval for the 1992 annual inspection of diesel generator G02 and the Service and Performance testing of the swing safety-related battery (D-305) will be deleted from pages 15.4.6-2 and 15.4.6-3. These notations are no longer applicable, and currently read as follows:

* The surveillance interval for the 1992 annual inspection of diesel generator G02 may be extended up to 6 months, not to exceed a total time between annual inspections of 18 months.

** Service and Performance testing to begin subsequent to installation of the swing safety-related battery (D-305) which is expected by the end of 1992.

Both notations are currently on page 15.4.6-2. Page 15.4.6-3 currently has only the notation relevant to the swing safety-related battery.

2.0 EVALUATION

Change 1: WEPCo has requested this change because of what they view as a potential conflict with TS 15.3.1.A.3.a(1) if neither reactor coolant loop is available for decay heat removal or 15.3.1.A.3.b if reactor coolant temperature is less than 140 °F. WEPCo could not determine a basis for the requirement that the breakers be in the test position. They note that the requirement appeared in the Technical Specifications as originally issued. They postulate that the test was originally done by placing the breakers in

the test position and that this became incorporated in the technical specifications as a requirement. Requirements subsequently added to Technical Specification 15.3.1.A.3.a(1) (See Amendments 66 and 71 for Units 1 and 2, respectively, dated November 8, 1982, and Amendments 82 and 86 for Units 1 and 2, respectively, dated February 2, 1984) create a potential conflict under some circumstances.

To ensure redundant decay heat removal capability, TS 15.3.1.A.3.a(1) requires that, when reactor coolant temperature is less than 350 °F and greater than 140 °F, at least two of the following decay heat removal methods listed shall be operable: reactor coolant loop A, reactor coolant loop B, residual heat removal loop A, and residual heat removal loop B. Placing the motor breakers for the residual heat removal pumps in the test position, as required by the current technical specification, renders those pumps inoperable and may impair the redundant decay heat removal capability.

Further, TS 15.3.1.A.3.b prescribes that, with reactor coolant temperature less than 140 °F, one residual heat removal loop may be out of service when the reactor vessel head is removed and the refueling cavity flooded. One of the two residual heat removal loops may be temporarily out of service to meet surveillance requirements.

This latter requirement effectively prohibits the safety injection system test as written when reactor coolant temperature is less than 140 °F since both RHR loops are out of service with the breakers in the test position. This conflict could be resolved, as proposed by the licensee, by allowing the surveillance to be performed while the breakers are racked in and operable.

Placing the breakers in the test position prevents the motors from starting upon receipt of a test signal. By allowing the surveillance to be performed without placing the breakers in the test position, the surveillance could be performed during routine operation of the residual heat removal system. If the surveillance were done with motors racked in and operable, the motors would indeed start during the surveillance.

In its application, WEPCo points out that proper response of the safety injection system can be demonstrated with the breakers in either position. Since the tests are to be performed during reactor shutdowns for refueling, starting a residual heat removal or safety injection pump poses no safety concern. The standard technical specification (SR 3.5.2.6) requires only a periodic (nominally at 18-month intervals) verification that each ECCS pump starts automatically on an actual or simulated actuation test signal. Thus the proposed change is consistent with the standard.

The staff finds change 1 acceptable.

Change 2: Three of the items proposed to be changed have no safety significance and are acceptable: relocating the requirement to a separate section; changing the nomenclature of the switching device; and adding descriptive material to the Bases. There has been no change in the switching apparatus since the initial installation. This change in the name of the device is for accuracy.

Changing the requirement that the test of the emergency lighting be performed during the test of the automatic start of the diesel generators is the more substantive aspect of change 2. There are two sources of power for emergency lighting. One of the loads powered by the emergency diesel generators is the emergency lighting transformer. This load is not placed on the bus by the automatic load sequencer. It must be loaded manually from the control room. Therefore, there appears to be no reason why testing of the emergency lighting powered from the emergency bus must be done in conjunction with the emergency diesel generator test. These amendments are not intended to grant any relief from requirements for testing, but only to provide warranted flexibility in testing emergency lighting.

In accordance with 10 CFR Part 50, Appendix R, Part III J., emergency lighting units with at least an 8-hour battery power supply shall be provided in all areas needed for operation of safe shutdown equipment and in access and egress routes thereto. Following the loss of all AC power, the power supply for these Appendix R lighting loads is transferred automatically to the batteries. It is important that this system be tested periodically. However, the transfer to the DC batteries appears not to be affected by or otherwise related to diesel generator testing and no increase in reliability of the Appendix R emergency lighting is achieved by performing such testing during the testing of the diesel generators.

The standard technical specifications do not include a specific requirement for testing emergency lighting.

Because testing of the emergency lighting is not affected by operation of the emergency diesel generators, the proposed amendments would have no impact to safety and are therefore acceptable.

Change 3:

WEPCo completed the tasks required by the two designated milestones. The second of the two items to be deleted also includes continuing requirements for record maintenance. Although the licensee would delete the record requirements from Technical Specifications, these requirements remain in effect because of their inclusion in 10 CFR 50.49(j). Since the licensee completed the specified tasks, the substantive aspects of the specification are no longer effective. Therefore, there is no safety significance to deleting the wording from the technical specifications. Change 3 is acceptable.

Administrative Improvement:

The notations relevant to the surveillance interval for the 1992 annual inspection of diesel generator G02 and the start of the Service and Performance testing subsequent to the installation of the swing safety-related battery (D-305) are no longer applicable. The licensee (Mr. Tom Malinowski) confirmed that the notations are no longer applicable in a conference call with the NRC (Mr. Tim McGinty, NRR/PDIII-3) on 8/2/93. The staff finds the deletion of these notes from the Technical Specifications an administrative improvement with no impact on safety, and therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (56 FR 43819). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. B. Samworth

Date: September 7. 1993