

August 24, 1989

Docket Nos. 50-266  
and 50-301

Mr. C. W. Fay, Vice President  
Nuclear Power Department  
Wisconsin Electric Power Company  
231 West Michigan Street, Room 308  
Milwaukee, Wisconsin 53201

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Dear Mr. Fay:

SUBJECT: AMENDMENT NOS. 124 AND 127 TO FACILITY OPERATING LICENSE NOS. DPR-24  
AND DPR-27 (TACS 73410 AND 73411)

The Commission has issued the enclosed Amendment Nos. 124 and 127 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated April 26, 1989.

These amendments clarify the reporting requirements for operation of the Overpressure Mitigating System (OMS) and delete two schedular commitments which have been implemented.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Warren H. Swenson, Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 124 to DPR-24
2. Amendment No. 127 to DPR-27
3. Safety Evaluation

DFol  
/1

cc w/enclosures:  
See next page

Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 7/19/89

PE/PDIII-3  
LKokajko/tg  
7/19/89

PM/PDIII-3  
Wswenson  
8/8/89

for: PD/PDIII-3  
JHannon  
8/9/89

OGC-WF1  
PAS  
8/16/89

see modification to Encl. Considered

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P PNU

CP-1 cc

Mr. C. W. Fay  
Wisconsin Electric Power Company

Point Beach Nuclear Plant  
Units 1 and 2

cc:

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Town Chairman  
Town of Two Creeks  
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Chairman  
Public Service Commission  
of Wisconsin  
Hills Farms State Office Building  
Madison, Wisconsin 53702

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
Office of Executive Director  
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799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Resident Inspector's Office  
U.S. Nuclear Regulatory Commission  
6612 Nuclear Road  
Two Rivers, Wisconsin 54241



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 26, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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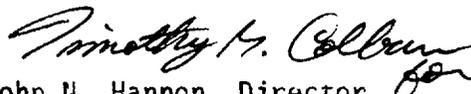
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 124, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 24, 1989



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY  
DOCKET NO. 50-301  
POINT BEACH NUCLEAR PLANT, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 26, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

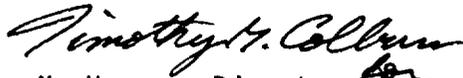
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 24, 1989

ATTACHMENT TO LICENSE AMENDMENT NOS.124 AND 127  
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27  
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

Page 15.6.9-4  
Table 15.3.5-1 (Page 2 of 2)  
Table 15.7.4-2 (Continued)

INSERT

Page 15.6.9-4  
Table 15.3.5-1 (Page 2 of 2)  
Table 15.7.4-2 (Continued)

#### 15.6.9.2 Unique Reporting Requirements

The following written reports shall be submitted to the Director, Office of Nuclear Reactor Regulation, USNRC:

##### A. Integrated Leak Rate Test

Each integrated leak test shall be the subject of a summary technical report, including results of the local leak rate tests and isolation valve leak rate tests since the last report. The report shall include analysis and interpretations of the results which demonstrate compliance with specified leak rate limits.

##### B. Poison Assembly Removal From Spent Fuel Storage Racks

Plans for removal of any poison assemblies from the spent fuel storage racks shall be reported and described at least 14 days prior to the planned activity. Such report shall describe neutron attenuation testing for any replacement poison assemblies, if applicable, to confirm the presence of boron material.

##### C. Overpressure Mitigating System Operation

In the event the overpressure mitigating system (power operated relief valves in the low temperature overpressure protection mode) or residual heat removal system relief valves are operated to relieve a pressure transient which, by licensee's evaluation, could have resulted in an overpressurization incident had the system not been operable, a special report shall be prepared and submitted to the Commission within 30 days. The report shall describe the circumstances initiating the transient, the effect of the system on the transient and any corrective action necessary to prevent recurrence.

Unit 1 - Amendment No. 55, 57, 58, 74, 78, 124  
 Unit 2 - Amendment No. 60, 61, 62, 79, 82, 127

TABLE 15.3.5-1 (Continued)  
(Page 2 of 2)

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>CHANNEL</u>	<u>SETTING LIMIT</u>
9.	Degraded Voltage (4.16 KV)	Disconnection of affected bus from offsite power	>3875 volts $\pm 2\%$ Time delay: less than 60 sec at 0-100% of voltage setting
10.	Loss of Voltage		
	a. 4.16 KV	Disconnection of affected bus from offsite power Start Diesel	a. 3220 volts $\pm 2\%$ Time Delay: $\leq 1$ sec $\pm 10\%$ at 0-100% of voltage setting
	b. 480 V	Load shedding	b. 256 volts $\pm 3\%$ Time delay: $\leq .75$ sec $\pm 10\%$ at 0 volts $< 3.5$ sec $\pm 20\%$ at 90% of voltage setting

TABLE 15.7.4-2 (Continued)

Unit 1 - Amendment No. 97, 124  
 Unit 2 - Amendment No. 101, 127

<u>Instrument Description</u>	<u>Channel Check</u>	<u>Calibrate</u>	<u>Functional Test</u>	<u>Source Check</u>
c. RE-305, Noble Gas (Purge Exhaust SPING - 1 per unit)	D	R	Q	M*
d. Iodine and Particulate Continuous Air Sampler	P/W	NA	NA	NA
e. Sampler Flow Rate Measuring Device	P/D	R	NA	NA
5. Fuel Storage and Drumming Area Ventilation Stack				
a. RE-221, Noble Gas (Drumming Area Vent Stack)	D	R	Q	M
b. RE-325, Noble Gas (Drumming Area SPING)	D	R	Q	M
c. Isokinetic Iodine and Particulate Continuous Air Sampling System	W	NA	NA	NA
6. Gas Stripper Building Ventilation System				
a. RE-224 Noble Gas	D	R	Q	M
b. Iodine and Particulate Continuous Air Sampler	W	NA	NA	NA
c. Sampler Flow Rate Measuring Device	W	R	NA	NA
7. Waste Gas Holdup System Explosive Gas Monitoring System				
a. Oxygen Monitor	D	Q**	Q	NA



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 124 AND 127 TO  
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY  
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated April 26, 1989, Wisconsin Electric Power Company (the licensee) submitted an application for amendments to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. The proposed amendments would revise Technical Specification (TS) 15.6.9.2.C to clarify the reporting requirements for operation of the Overpressure Mitigating System (OMS). Additionally, the amendments would delete two schedular commitments as found in footnotes in TS Table 15.3.5-1 (item 10) and TS Table 15.7.4-2 (item 7.a)

2.0 EVALUATION

The licensee proposed to modify a unique reporting requirement in TS 15.6.9.2.C, "Overpressure Mitigating System Operation." This item requires that a special report be submitted to the Commission within 30 days of an event which, by licensee's evaluation, could have resulted in an overpressurization incident had the OMS not been operable. Until recently, the licensee had interpreted this to apply only to the actuation of the pressurizer power operated relief valves (PORV) in the low temperature overpressure protection (LTOP) mode. However, as a result of an event reported in a letter to the NRC dated February 9, 1989, it was discovered that the NRC's Safety Evaluation Report (SER) dated May 20, 1980, accompanying the approval of this unique reporting requirement, contained an additional reportable item. The SER directed that the special report be submitted not only for PORV LTOP operation, but also for actuation of the residual heat removal (RHR) system relief valves if such actuation would have mitigated an overpressurization incident. The SER also recognized the RHR system relief valves as a diverse relief system at Point Beach Nuclear Plant since the RHR system is not automatically isolated during a pressure transient. As a result of this discovery, the licensee proposed to revise the specification to provide assurance that operation of either the PORV's in the LTOP mode or the RHR system relief valves are reported as required.

The licensee also proposed to remove two schedular commitments found in the footnotes of two TS tables. The first commitment is contained in Table 15.3.5-1 and is related to the setting limit for item 10, "Loss of Voltage." The associated relays have been installed as required, making this footnote no longer appropriate. The second commitment is found in Table 15.7.4-2 and is in regards to item 7.a, "Oxygen Monitor." The oxygen monitor has been installed as required, making this footnote no longer appropriate.

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The staff concludes that the proposed changes to the Technical Specifications are entirely administrative in nature, reflecting actual reporting requirements and the completion of changes previously committed to by the licensee. Based on the information presented above, the staff concludes that the proposed changes to the Technical Specifications are acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also involves changes in record-keeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence E. Kokajko

Dated: August 24, 1989