

February 23, 1990

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

DISTRIBUTION:

<u>Docket Files</u>	DHagan
NRC PDR	EJordan
Local PDR	ARM/LFMB
PDIII-3 r/f	Ghill(8)
PDIII-3 Gray	WandaJones
JZwolinski	JCalvo
PKreutzer	ACRS(10)
WSwenson	GPA/PA
JHannon	BJones, RSB
OGC-WF1	

Dear Mr. Fay:

SUBJECT: AMENDMENT NOS. 126 AND 130 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 (TACS 68862/63)

The Commission has issued the enclosed Amendment Nos. 126 and 130 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated July 6, 1988 as supplemented November 1, 1989.

These amendments revise Technical Specification 15.5.4.2 relating to fuel storage. The amendments increase the allowable U-235 content for optimized fuel assemblies (OFA) to 46.8 grams per axial centimeter and permit the use of axial fuel blankets. The allowable U-235 content for the OFA fuel assemblies corresponds to an enrichment of 4.75 weight percent. The U-235 content permitted for standard fuel assemblies remains unchanged.

Copies of the Safety Evaluation and of the Notice of Issuance are also enclosed. The notice of issuance has been forwarded to the Office of the Federal Register for publication.

Sincerely,

/s/

Warren H. Swenson, Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 126 to DPR-24
2. Amendment No. 130 to DPR-27
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

Office: LA/PDIII-3
Surname: PKreutzer
Date: 2/13/90

PM/PDIII-3
WSwenson/tg
2/21/90

PD/PDIII-3
JHannon
2/21/90

RSB
BJones
2/21/90

OGC
BJones
2/22/90

CIP-1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 23, 1990

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

SUBJECT: AMENDMENT NOS. 126 AND 130 TO FACILITY OPERATING LICENSE NOS. DPR-24
AND DPR-27 (TACS 68862/63)

The Commission has issued the enclosed Amendment Nos. 126 and 130 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated July 6, 1988 as supplemented November 1, 1989.

These amendments revise Technical Specification 15.5.4.2 relating to fuel storage. The amendments increase the allowable U-235 content for optimized fuel assemblies (OFA) to 46.8 grams per axial centimeter and permit the use of axial fuel blankets. The allowable U-235 content for the OFA fuel assemblies corresponds to an enrichment of 4.75 weight percent. The U-235 content permitted for standard fuel assemblies remains unchanged.

Copies of the Safety Evaluation and of the Notice of Issuance are also enclosed. The notice of issuance has been forwarded to the Office of the Federal Register for publication.

Sincerely,

A handwritten signature in black ink that reads "Warren H. Swenson". The signature is written in a cursive style with a large, sweeping flourish at the end.

Warren H. Swenson, Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 126 to DPR-24
2. Amendment No. 130 to DPR-27
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

Mr. C. W. Fay
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Units 1 and 2

cc:

Mr. Bruce Churchill, Esq.
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N.W.
Washington, DC 20037

Mr. James J. Zach, Manager
Point Beach Nuclear Plant
Wisconsin Electric Power Company
6610 Nuclear Road
Two Rivers, Wisconsin 54241

Town Chairman
Town of Two Creeks
Route 3
Two Rivers, Wisconsin 54241

Chairman
Public Service Commission
of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
Office of Executive Director
for Operations
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Resident Inspector's Office
U.S. Nuclear Regulatory Commission
6612 Nuclear Road
Two Rivers, Wisconsin 54241



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
DOCKET NO. 50-266
POINT BEACH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 126
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated July 6, 1988 as supplemented November 1, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9003010302 900223
PDR ADOCK 05000266
P PDC

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 126, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Timothy M. Colburn

John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 23, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
DOCKET NO. 50-301
POINT BEACH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated July 6, 1988 as supplemented November 1, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Timothy H. Callum

John N. Hannon, Director *for*
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 23, 1990

ATTACHMENT TO LICENSE AMENDMENT NOS. 126 AND 130
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the page identified below and inserting the enclosed page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

REMOVE

15.5.4-1

INSERT

15.5.4-1

15.5.4 FUEL STORAGE

Applicability

Applies to the capacity and storage arrays of new and spent fuel.

Objective:

To define those aspects of fuel storage relating to prevention of criticality in fuel storage areas.

Specification

1. The new fuel storage and spent fuel pool structures are designed to withstand the anticipated earthquake loadings as Class I structures. The spent fuel pool has a stainless steel liner to ensure against loss of water.
2. The new and spent fuel storage racks are designed so that it is impossible to store assemblies in other than the prescribed storage locations. The fuel is stored vertically in an array with sufficient center-to-center distance between assemblies to assure $K_{eff} < 0.95$ with the storage pool filled with unborated water and with the fuel loading in the assemblies limited to 44.8 grams of U-235 per axial centimeter of standard fuel assembly and 46.8 grams of U-235 per axial centimeter of OFA fuel assembly with or without axial blanket loadings. An inspection area shall allow rotation of fuel assemblies for visual inspection, but shall not be used for storage.
3. The spent fuel storage pool shall be filled with borated water at a concentration of at least 1800 ppm boron whenever there are spent fuel assemblies in the storage pool.
4. Except for the two storage locations adjacent to the designated slot for the spent fuel storage rack neutron absorbing material surveillance specimen irradiation, spent fuel assembly storage locations immediately adjacent to the spent fuel pool perimeter or divider walls shall not be occupied by fuel assemblies which have been subcritical for less than one year.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 126 AND 130 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated July 6, 1988 as supplemented November 1, 1989, Wisconsin Electric Power Company (the licensee) applied for amendments to Facility Operating Licenses DPR-24 and DPR-27 of the Point Beach Nuclear Plant to revise the maximum fuel enrichment limit specified in Technical Specification (TS) 15.5.4.2. The application also modified the wording of the TS to permit the use of axial fuel blankets.

Facility Operating Licenses DPR-24 and DPR-27 for the Point Beach Nuclear Plant were amended on October 5, 1984 (Amendments 86 and 90, respectively) to include the use of OFA fuel with an allowable U-235 content limited to 39.4 grams per axial centimeter (equivalent to 4.0 weight percent). The effect of the new OFA fuel on criticality, spent fuel cooling requirements, and radiological consequences as well as the gamma heating effects were evaluated and found to be acceptable. The facility operating licenses were amended again on April 14, 1989 (Amendments 117 and 120, respectively), raising the allowable U-235 content per axial centimeter to 40.0 grams for OFA fuel.

2.0 EVALUATION

The licensee requested that the U-235 loading limit specified in TS 15.5.4.2 for fuel storage in the new fuel storage vault and the spent fuel storage pool be revised to 46.8 grams per axial centimeter for optimized fuel assemblies (OFA) and that the use of axial fuel blankets be permitted. The U-235 loading level is not a parameter that is considered in accident analyses for operations of the Point Beach Nuclear Plant. Rather, the safety margins are maintained by other limits such as power, power distribution, reactivity coefficients, burnup, etc., which are verified to be acceptable by cycle-specific reload analyses.

2.1 FUEL STORAGE

The water in the spent fuel storage pool at the Point Beach Nuclear Plant contains at least 1800 parts per million of boron. The spent fuel storage racks consist of an array of individual square cross-section, rectangular cylinders with an outer dimension of approximately 10 inches and a length of about 14 feet. The racks are designed so that it is impossible to store fuel

assemblies within the racks in other than a valid storage position, thereby ensuring the necessary spacing between the assemblies. The rectangular cylinders are edge-welded to form a honeycomb structure. Two poison assemblies are inserted into contiguous poison compartments formed inside each of the storage cylinders.

Each poison assembly supports two Boraflex sandwiches. Each sandwich consists of a Boraflex sheet between two stainless steel plates. There are two Boraflex sandwiches between any two stored fuel assemblies in the spent fuel storage pool.

Criticality analyses of the Point Beach new and spent fuel storage racks for storage of Westinghouse OFA fuel were originally performed for the licensee by Pickard, Lowe and Garrick, Inc. (PLG), in support of their application to use OFA fuel at the Point Beach Nuclear Plant. The NRC issued amendments approving the use and storage of OFA fuel with a U-235 content limited to 39.4 grams per axial centimeter as noted above.

The original PLG analyses utilized the LEOPARD and PDQ-7 computer programs. LEOPARD and PDQ-7 calculational accuracies were verified by means of benchmark comparisons with critical assembly experiments, and conservative techniques were used for the determination of the infinite neutron multiplication factor, k-effective. The calculations were performed with the same methods that had been used and approved previously for the new and spent fuel racks at Point Beach. The licensee has used the same methodology to determine k-effective for higher enrichment fuel. For OFA fuel with a U-235 content of 4.75 weight percent, the maximum k-effective for the spent fuel racks was calculated to be 0.9406, including all biases and uncertainties. This maximum value of k-effective is acceptable because it is less than the 0.95 regulatory limit. Furthermore, it should be noted that all neutron multiplication factors were calculated assuming a spent fuel pool filled with unborated water.

The licensee addressed the effects of higher enrichment fuel and increased burnup on spent fuel pool cooling and gamma heating. The original OFA analysis provided by the licensee in support of license amendments 86 and 90 to Facility Operating Licenses DPR-24 and DPR-27, respectively, quantified those effects for spent fuel with an initial enrichment of up to 4.0 weight percent U-235. The licensee has completed further analysis which demonstrates conservative changes in the those parameters of interest for the more highly enriched fuel at increased burnup. As a result, the licensee concluded that the original analysis completed for the OFA fuel bounds the expected effects resulting from the proposed increase in allowable U-235 content for OFA fuel.

The new fuel storage vault is designed to hold new fuel assemblies in specially constructed dry storage racks. The center-to-center spacing of the assemblies in the new fuel storage vault is 20 inches.

Similar analyses were performed for the new fuel storage vault in the flooded state. For the new fuel storage vault, all calculations were performed using new Westinghouse OFA fuel with an enrichment of 5.5 weight percent. The use of 5.5 weight percent fuel is conservative. In addition to the baseline case for a flooded cavity, calculations were performed for elevated temperatures and for mist conditions with water densities ranging from 3 to 80 percent of maximum water density. The LEOPARD/PDQ model bias for these cases is 0.0071 delta-k and the calculational uncertainty is 0.0029 delta-k. In addition, the increase in k-effective due to fuel position uncertainties and due to the maximum fuel pellet density were taken into account. The combination of these biases and uncertainties yielded a maximum expected value of k-effective for the new fuel storage racks of 0.9221 for the fully flooded case. This value of k-effective is acceptable because it is less than the 0.95 regulatory limit.

The licensee presently has no plans to utilize axial fuel blankets at Point Beach but wishes to have the operational flexibility to do so. The licensee has confirmed the staff's understanding that its use of an optional axial-zoned core-loading scheme refers to the use of a single enrichment throughout the active portion of the fuel assembly with axial blankets of natural (non-enriched) fuel above and below the active portion of the fuel assembly. The storage of such fuel in either the new fuel vault or the spent fuel storage pool is not a concern since the replacement of the more highly enriched fuel with non-enriched fuel at the top and bottom will further reduce the neutron multiplication factor, k-effective. As a result, the use of OFA fuel with axial fuel blankets is acceptable.

2.2 DESIGN BASIS FUEL HANDLING ACCIDENT

The licensee has requested authorization to increase fuel enrichment to 4.75 weight percent of U-235 with an expected average fuel burnup of 45,000 MWD/MT. Both the staff and the licensee have evaluated the potential impact of this change on the radiological assessment of design basis accidents (DBAs) which were previously analyzed in the licensing of the Point Beach Units 1 and 2 nuclear power plants.

The staff reviewed the licensee's submittals and also reviewed a publication which was prepared for the NRC entitled, "Assessment of the Use of Extended Burnup Fuel in Light Water Reactors," NUREG/CR 5009, February 1988. The NRC contractor, the Pacific Northwest Laboratory (PNL) of Battelle Memorial Institute, examined the changes that could result in the NRC design basis accident (DBA) assumptions, as described in appropriate sections of the staff's Standard Review Plan and/or Regulatory Guides, that could result from the storage and handling of extended burnup fuel, which would require higher initial enrichment. The staff agrees with PNL that the only DBA that could be affected by the use of extended burnup fuel, in even a minor way, would be the potential thyroid doses that could result from a postulated design basis fuel handling accident. PNL estimates that I-131 fuel gap activity in the peak fuel rod with 60,000 MWD/MT burnup (5.292 initial weight percent U-235) could be as high as 12%. This value is approximately

20% higher than the value normally used by the staff in evaluating fuel handling accidents (Regulatory Guide 1.25, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facilities for Boiling and Pressurized Water Reactors").

The staff, therefore, reevaluated the fuel handling accident for Point Beach Units 1 and 2 with an increase in iodine gas activity in the fuel damaged in a postulated fuel handling accident. Table 1 presents the conservatively analyzed fuel handling accident doses presented in the staff's Safety Evaluation dated April 4, 1979 related to Amendments 35 and 41 to license Nos. DPR-24 and DPR-27, respectively. These estimates were based on a presumed fuel peaking factor of 1.65. The licensee's Final Safety Analysis Report (FSAR), updated through October 1, 1988, assumes a peaking factor of 1.8 (p. 14.2.1-11). The current staff dose estimates result from multiplying its previous estimates by $1.2 \times 1.8/1.65 = 1.31$, to account for these two factors. The results are also shown in Table 1. The resulting doses are small fractions of the applicable regulatory requirements of 10 CFR Part 100.

The staff concludes that the bounding doses potentially increased are the thyroid doses at the Exclusion Area and Low Population Zone boundaries resulting from postulated fuel handling accidents, that these doses remain well within the 300 rem thyroid exposure guideline values set forth in 10 CFR Part 100, that the small calculated increase is not significant and that the Technical Specification change requested by the licensee is acceptable.

2.3 OTHER CONSIDERATIONS

On April 4, 1979, the NRC staff issued a safety evaluation report approving the surveillance program for Boraflex to be used as a neutron attenuation material in the spent fuel storage pools at the Point Beach Nuclear Plant. The surveillance program would verify the continued integrity of the Boraflex material in the spent fuel storage racks. On February 11, 1987, the licensee reported to the NRC the results of the Boraflex examinations performed in 1985 and 1986. Ten Boraflex coupons were found to have significant decreases in sample thickness, width, and weight, and were fragile. Nevertheless, the neutron attenuation capability of these coupons was not significantly reduced. Further, the full length Boraflex inserts examined showed no degradation other than some discoloration along the edges of the irradiated insert.

As a result of the observed degradations in the Boraflex coupons, the licensee proposed to replace the existing Boraflex surveillance procedure with a new surveillance program. On April 13, 1989, the licensee submitted for NRC approval a revised proposal to modify the Boraflex surveillance program. Further, by letter dated November 1, 1989, the licensee described corrective actions to be taken upon finding degraded Boraflex to ensure subcriticality of the stored fuel assemblies. On February 21, 1990, the NRC staff issued a safety evaluation

approving the revised Boraflex surveillance program. The safety evaluation concluded that the proposed surveillance program was sufficient to detect deterioration of the physical integrity of the Boraflex that might lead to the loss of neutron attenuation capability during the design life of the spent fuel storage racks. Further, the staff concluded that should significant loss of Boraflex neutron attenuation be found, the licensee can take corrective actions to ensure subcriticality of the stored fuel assemblies.

On November 21, 1989, the NRC issued Bulletin 89-03, "Potential Loss of Required Shutdown Margin During Refueling Operations." The bulletin noted that as a result of the longer fuel operating cycles, utilities have been increasing the enrichment of reload fuel. Some of these fresh reload fuel assemblies may be highly reactive under certain refueling conditions. Although analyses are performed for PWRs to confirm that the refueling boron concentration is sufficient to maintain the required shutdown margin for the final core configuration, the staff was concerned that these analyses may not be sufficient to assure that the shutdown margin will be maintained for all intermediate fuel assembly positions.

The licensee responded to Bulletin 89-03 with a letter dated January 11, 1990. In their response, the licensee indicated that actions, procedures, and training necessary to comply with the actions requested by Bulletin 89-03 would be completed prior to shutdown for the Point Beach Unit 1 refueling outage. This outage is currently scheduled to begin on March 30, 1990. By letter dated February 15, 1990, the NRC staff notified the licensee that its response was satisfactory.

3.0 FINDINGS

The NRC staff has reviewed the request by the Wisconsin Electric Power Company to increase the allowable U-235 content for optimized fuel assemblies (OFA) to 46.8 grams per axial centimeter and to permit the use of axial fuel blankets. Based on this review, the staff has concluded that the storage and use of such fuel at the Point Beach Nuclear Plant is acceptable and that the Technical Specifications submitted for this license amendment accurately reflect the modifications necessary to accommodate future fuel reloads. Therefore, the proposed changes to TS 15.5.4.2 are acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact has been prepared and published in the Federal Register on February 23, 1990 (55 FR 6564). Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of these amendments will not have a significant effect on the quality of the human environment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Martin
W. Swenson

Dated: February 23, 1990

Attachment: Table 1

Table 1

Estimated Doses as a Consequence of DBA Fuel Handling Accident

	<u>Exclusion Area Boundary*</u>	
	<u>Thyroid</u>	<u>Whole Body</u>
April 4, 1979 NRC Staff Estimate	36 rem	0.13 rem
Current NRC Staff Estimate	47 rem	0.17 rem
10 CFR Part 100	300 rem	25 rem

*Low Population Zone doses are less than Exclusion Area Boundary doses

UNITED STATES NUCLEAR REGULATORY COMMISSIONWISCONSIN ELECTRIC POWER COMPANYDOCKET NOS. 50-266 AND 50-301NOTICE OF ISSUANCE OF AMENDMENTS TOFACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 126 to Facility Operating License No. DPR-24 and Amendment No. 130 to Facility Operating License No. DPR-27, issued to the Wisconsin Electric Power Company (the licensee), which revised the Technical Specifications for operation of the Point Beach Nuclear Plant Units 1 and 2, located in Manitowoc County, Wisconsin. The amendments were effective as of the date of issuance.

The amendments modified the Technical Specifications to increase the allowable U-235 content for optimized fuel assemblies (OFA) to 46.8 grams per axial centimeter and permit the use of axial fuel blankets. The allowable U-235 content for the OFA fuel assemblies corresponds to an enrichment of 4.75 weight percent. The U-235 content permitted for standard fuel assemblies remains unchanged.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

9003010307 900223
PDR ADCK 05000266
P PDC

Notice of Consideration of Issuance of Amendments and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on February 23, 1989 (54 FR 7900). No request for a hearing or petition for leave to intervene was filed following this notice. By letter dated November 1, 1989, the licensee submitted clarifying information in response to the NRC's request dated October 6, 1989. That letter clarified information previously provided and did not revise the application for license amendment.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendments dated July 6, 1988, and supplemented November 1, 1989, (2) Amendment No. 126 to License No. DPR-24, (3) Amendment No. 130 to License No. DPR-27, and (4) the Commission's related Safety Evaluation dated February 23, 1990 and (5) the Environmental Assessment dated February 23, 1990. All of these items are available for public inspection at the Commission's Public Document Room, Gelman Building 2120 L Street N.W., Washington, D.C., and at the Joseph P. Mann Library, 1516 Sixteenth Street, Two Rivers, Wisconsin. A copy of items (2), (3), (4) and (5), may be obtained upon request addressed to the U.S.

Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director,
Division of Reactor Projects III, IV, V and Special Projects.

Dated at Rockville, Maryland this 23rd day of February 1990.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy G. Colburn, Acting Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation