

Docket Nos. 50-266
and 50-301

May 26, 1992

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

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Dear Mr. Link:

SUBJECT: AMENDMENT NOS. 131 AND 135 TO FACILITY OPERATING LICENSE NOS. DPR-24
AND DPR-27 (TAC NOS. M81743 AND M81744)

The Commission has issued the enclosed Amendment Nos. 131 and 135 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications (TS) in response to your application dated September 13, 1991.

These amendments revise TS Section 15.3.1.B., Reactor Coolant System, Pressure/Temperature Limits, and its Bases, and removes TABLE 15.3.1-1 (Unit No. 1) and TABLE 15.3.1-2 (Unit No. 2), Reactor Vessel Surveillance Capsule Removal Schedule.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

original signed by

Robert B. Samworth, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 131 to DPR-24
2. Amendment No. 135 to DPR-27
3. Safety Evaluation

*See Previous Concurrence

cc w/enclosures:
See next page

LA: PD3-3: DRPW	*PM: PD3-3: DRPW
PKreutzer	JLombardo/bj
5/26/92	1/92

RS
PM: PD3-3: DRPW
RSamworth
5/26/92

*BC: OTSB: DOEA
CGrimes
5/13/92

D: PD3-3: DRPW	*OGC-WF
JHannon	
5/26/92	5/14/92

DOCUMENT NAME WP: a:PB81743.AMD

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Mr. Robert E. Link
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 131
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 13, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 131 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert B. Samworth, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: May 26, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 135
License No. DPR-27

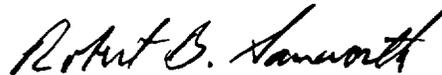
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 13, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 135 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert B. Samworth, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: May 26, 1992

ATTACHMENT TO LICENSE AMENDMENT NOS. 131 AND 135
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
15.3.1-4	15.3.1-4
15.3.1-8	15.3.1-8
Table 15.3.1-1 (Unit 1)	--
Table 15.3.1-2 (Unit 2)	--

B. Pressure/Temperature Limits

Specification:

1. The Reactor Coolant System temperature and pressure shall be limited in accordance with the limit lines shown in Figure 15.3.1-1 and 15.3.1-2 during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:
 - a. A maximum heatup of 100°F in any one hour,
 - b. A maximum cooldown of 100°F in any one hour, and
 - c. An average temperature change of $\leq 10^\circ\text{F}$ per hour during inservice leak and hydrostatic testing operations.
2. The secondary side of the steam generator will not be pressurized above 200 psig if the temperature of the steam generator vessel shell is below 70°F.
3. The pressurizer temperature shall be limited to:
 - a. A maximum heatup of 100°F in any one hour and a maximum cooldown of 200°F in any one hour, and
 - b. A maximum spray water temperature differential between the pressurizer and spray fluid of not greater than 320°F.
4. The reactor vessel irradiation surveillance specimens are removed and examined, according to NRC approved schedules, to determine changes in material properties. The results of these examinations shall be considered in the evaluation of the prediction method to be used to update Figures 15.3.1-1 and 15.3.1-2. Revised figures shall be provided to the Commission at least sixty (60) days before the calculated exposure of the applicable reactor vessel exceeds the exposure for which the figures apply.

Unit 1 Amendment No. ~~24, 27, 125~~, 131

Unit 2 Amendment No. ~~28, 33, 128~~, 135 15.3.1-4

The actual temperature shift of the vessel material will be established periodically during operation by removing and evaluating reactor vessel material irradiation surveillance specimens installed near the inside wall of the reactor vessel in the core area. Since the neutron spectra at the irradiation samples and vessel inside radius are identified by a specified lead factor, the measured temperature shift for a sample is an excellent indicator of the effects of power operation on the adjacent section of the reactor vessel. If the experimental temperature shift (at the 30 ft-1b level) does not substantiate the predicted shift, new prediction curves and heatup and cooldown curves must be developed.

The pressure-temperature limit lines shown on Figure 15.3.1-1 for reactor criticality and for inservice leak and hydrostatic testing have been provided to assure compliance with the minimum temperature requirements of Appendix G to 10 CFR 50 for reactor criticality and for inservice leak and hydrostatic testing.

The spray should not be used if the temperature difference between the pressurizer and spray fluid is greater than 320°F. This limit is imposed to maintain the thermal stresses at the pressurizer spray line nozzle below the design limit.

The temperature requirements for the steam generator correspond with the measured NDT for the shell.

The reactor vessel materials surveillance capsule removal schedules have been developed based upon the requirements of the Code of Federal Regulations, Title 10, Part 50, Appendix H, and with consideration of ASTM Standard E-185-82. When the capsule lead factors are considered, the scheduled removal dates accommodate the weld data needs of all the participants in the Babcock and Wilcox Master Integrated Reactor Vessel Surveillance Program. Additionally, the schedule will provide plate/forging material data as well as fluence data corresponding to the expiration of the current licenses and of any future license extensions.

References

- (1) FSAR, Section 4.1.5
- (2) Westinghouse Electric Corporation, WCAP-10638
- (3) Westinghouse Electric Corporation, WCAP-8743
- (4) Westinghouse Electric Corporation, WCAP-8738
- (5) Babcock & Wilcox, BAW 1803
- (6) Regulatory Guide 1.99, Revision 2



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 131 AND 135 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated September 13, 1991, the Wisconsin Electric Power Company (the licensee), requested changes to Technical Specification (TS) 15.3.1-B, Reactor Coolant System, Pressure/Temperature Limits, and the associated Bases. The proposed changes would remove Tables 15.3.1-1 and 15.3.1-2, the reactor vessel materials surveillance capsule removal schedules for Units 1 and 2 respectively, from the TS. The licensee proposes to incorporate these tables into the Point Beach Final Safety Analysis Report (FSAR) in the next revision. In addition, the licensee will include any subsequent NRC-approved revisions to this schedule in an update of the FSAR. The inclusion of the withdrawal schedule in the FSAR provides a source for this information that is readily available as a reference for NRC inspectors and other staff use. Finally, the surveillance requirements for removing material specimens remain unchanged except for the removal of the reference to Tables 15.3.1-1 and 15.3.1-2.

2.0 EVALUATION

Appendix H to 10 CFR Part 50 requires that a reactor vessel material surveillance program be maintained in order to prevent brittle fracture of the reactor vessel. Section II.B.3 of this appendix requires that the associated withdrawal schedules be approved prior to their implementation. Having these schedules in the TS duplicates the regulatory controls placed on them by Appendix H. Surveillance requirements will still require that, following removal, these specimens be examined to determine any changes in material properties. The results of these examinations will be used to update temperature and pressure limits. Therefore, there is no need for these schedules to be included in the TS.

Accordingly, the licensee proposes to remove Tables 15.3.1-1 and 15.3.1-2 from the TS and incorporate them into the FSAR in its next revision.

This amendment will not result in any change in specimen withdrawal schedules, and the actions taken subsequent to specimen withdrawal will remain unchanged. The licensee has stated that no physical change to the facility, its systems, or its operation will be made. The proposal changes are consistent with the guidance in Generic Letter 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications." Controls for governing future changes to the specimen withdrawal schedules shall be provided by 10 CFR, Part 50, Appendix H. The staff has reviewed the proposed changes to TS 15.3.1-B and Bases, and finds them to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (57 FR 4496). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Lombardo

Date: May 26, 1992