

July 31, 1992

Docket Nos. 50-266
and 50-301

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

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Dear Mr. Link:

SUBJECT: AMENDMENT NOS.133 AND 137 TO FACILITY OPERATING LICENSE NOS. DPR-24
AND DPR-27 (TAC NOS. M79954 AND M79955)

The Commission has issued the enclosed Amendment Nos.133 and 137 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated March 1, 1991 and supplemented December 6, 1991.

These amendments revise TS 15.4.8, Auxiliary Feedwater System, by changing sections 15.4.8.1.a and 15.4.8.1.b to require each auxiliary feedwater (AFW) pump to be started quarterly, and by providing the basis for TS 15.4.8 for this change. In addition TS15.3.4, Steam and Power Conversion System, is revised by changing section 15.3.4.c.2 to clarify the AFW pump out-of-service limitations.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by

Robert B. Samworth, Sr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.133 to DPR-24
2. Amendment No.137 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

LA: PDIII-3:DRPW
PKreutzer
7/13/92

PM: PD3-3:DRPW
JLombardo
7/13/92

PM: PDIII-3:DRPW
RSamworth
7/13/92

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CMcCracken
7/14/92

D: PDIII-3:DRPW
JHannon
7/14/92

OGC-OWF
7/21/92

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P PDR

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

July 31, 1992

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and 50-301

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Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, Wisconsin 53201

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The Commission has issued the enclosed Amendment Nos. 133 and 137 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications (TS) in response to your application dated March 1, 1991 and supplemented December 6, 1991.

These amendments revise TS 15.4.8, Auxiliary Feedwater System, by changing sections 15.4.8.1.a and 15.4.8.1.b to require each auxiliary feedwater (AFW) pump to be started quarterly, and by providing the basis for TS 15.4.8 for this change. In addition TS 15.3.4, Steam and Power Conversion System, is revised by changing section 15.3.4.c.2 to clarify the AFW pump out-of-service limitations.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "R. B. Samworth".

Robert B. Samworth, Sr., Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 133 to DPR-24
2. Amendment No. 137 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Robert E. Link
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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Two Rivers, Wisconsin 54241



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 1, 1991 and supplemented December 6, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 133 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert B. Samworth, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: July 31, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated March 1, 1991 and supplemented December 6, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 137 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert B. Samworth, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: July 31, 1992

ATTACHMENT TO LICENSE AMENDMENT NOS. 133 AND 137
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

15.3.4-2a
15.4.8-1
15.4.8-2

INSERT

15.3.4-2a
15.4.8-1
-

2. Single Unit Operation - One of the three operable auxiliary feedwater pumps associated with a unit may be out-of-service for the below specified times. The turbine driven auxiliary feedwater pump may be out-of-service for up to 72 hours. If the turbine driven auxiliary feedwater pump cannot be restored to service within that 72-hour time period, the reactor shall be in hot shutdown within the next 12 hours. Either one of the two motor driven auxiliary feedwater pumps may be out-of-service for up to 7 days. If the motor driven auxiliary feedwater pump cannot be restored to service within that 7-day period, the operating unit shall be in hot shutdown within the next 12 hours.

Basis

A reactor shutdown from power requires removal of core decay heat. Immediate decay heat removal requirements are normally satisfied by the steam by pass to the condenser. Therefore, core decay heat can be continuously dissipated via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by operation of the turbine cycle feedwater system.

The eight main steam safety valves have a total combined rated capability of 6,664,000 lbs/hr. The total full power steam flow is 6,620,000 lbs/hr, therefore eight (8) main steam safety valves will be able to relieve the total full power steam flow if necessary.

In the unlikely event of complete loss of electrical power to the station, decay heat removal would continue to be assured for each unit by the availability of either the steam-driven auxiliary feedwater pump or one of the two motor-driven auxiliary steam generator feedwater pumps, and steam discharge to the atmosphere via the main steam safety valves or atmospheric relief valves. One motor-driven auxiliary feedwater pump can supply sufficient feedwater for removal of decay heat from a unit. The minimum amount of water in the condensate storage tanks ensures the ability to maintain each unit in a hot shutdown condition for at least one hour concurrent with a loss of all AC power.

15.4.8 AUXILIARY FEEDWATER SYSTEM

Applicability

Applies to periodic testing requirements of the turbine-driven and motor-driven auxiliary feedwater pumps.

Objective

To verify the operability of the Auxiliary Feedwater System and its ability to respond properly when required.

Specification

1. a. Each motor driven auxiliary feedwater pump will be started quarterly.
 - b. Each steam turbine driven auxiliary feedwater pump will be started quarterly provided steam is available.
 - c. The auxiliary feedwater pumps discharge valves and the service water supply valves on the suction side will be tested by operator action monthly.
2. These tests shall be considered satisfactory if control board indication and subsequent visual observation of the equipment demonstrate that all components have operated properly.

Basis

The quarterly testing of the auxiliary feedwater pumps will verify their operability. The quarterly test of the steam driven pumps will be a fast start test with no prior warmup. Proper functioning of the steam turbine admission valves and the feedwater pumps start will demonstrate the integrity of the steam driven pump. Verification of correct operation will be made both from instrumentation within the main control room and direct visual observation of the pumps.

Reference

FSAR - Sections 10.4 FSAR - Section 14.1.7 FSAR - Section 14.2.5



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 133 AND 137 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated March 1, 1991, the Wisconsin Electric Power Company (the licensee) requested changes to Technical Specification (TS) 15.4.8, Auxiliary Feedwater System, and associated Bases, for the Point Beach Nuclear Plant, Units 1 and 2. The proposed changes to sections 15.4.8.1.a and 15.4.8.1.b would require each auxiliary feedwater (AFW) pump to be started quarterly instead of monthly. The proposed change to the Bases establishes the quarterly start of each steam turbine-driven AFW pump as a fast start without prior warm-up.

By letter dated December 6, 1991, the licensee requested a change to TS 15.3.4, Steam and Power Conversion System, to clarify section 15.3.4.c.2, AFW pump out-of-service limitations.

2.0 EVALUATION

TS 15.4.8.1.a now requires that each motor-driven AFW pump be started monthly. The licensee has been performing these monthly tests since the units began operation in 1970 and 1972 for Point Beach Units 1 and 2, respectively. In all of these tests, as well as starts resulting from automatic system actuations, they have not experienced any failures of the pumps to start on demand, thus demonstrating a high degree of reliability.

TS 15.4.8.1.b now requires that each steam turbine-driven AFW pump be started monthly, provided steam is available. This monthly test is conducted by first blowing down the steam supply lines to remove accumulated water and then slowly bringing the pump to full speed over a 2-minute period. While the test demonstrates that the pump is operable, it does not test the pump under the conditions and configurations expected during an actual demand, i.e., it does not demonstrate cold, fast start capability.

The licensee has evaluated the issue of performing a cold, fast start of the turbine-driven AFW pump as part of its review of INPO Significant Operating Event Report (SOER) 86-01, "Reliability of Auxiliary Feedwater Systems." Recommendation 3 of that SOER suggests Terry turbine systems, such as are installed in the steam turbine-driven AFW pump at Point Beach Nuclear Plant, be subjected to periodic cold, fast start tests. Commencing in early 1990, the monthly test required by TS 15.4.8.1.b was conducted once each quarter as

a fast start test without prior warm-up of the steam supply piping and the pump. To date, four cold, fast starts on each of the steam turbine-driven AFW pumps for each unit have been conducted successfully. Prior to 1990 and twice each quarter in 1990, normal monthly tests have been performed. Throughout all those tests, as well as automatic starts resulting from actual system demands, no failures of the pumps to start have been experienced.

This change modifies the frequency of required starting of the AFW pumps and, in the case of the steam turbine-driven pump, requires this start at conditions more representative of those expected during an actual demand to start. Since only the testing frequency and conditions will be changed, there will be no physical change to the facility, its systems, or its operating procedures. Based on pump testing history, the present monthly testing has never indicated a pump failure, and the pumps have never failed to start in response to an actual demand, indicating a high degree of reliability. In addition, the quarterly interval is based on ASME Section XI requirements which provide adequate assurance of pump operability. The staff has reviewed the proposed changes to TS 15.4.8.1.a and 15.4.8.1.b and Bases, and finds them to be acceptable.

The intent of TS 15.3.4.C.2, Single Unit Operation, is to permit, as discussed in TS 15.3.0, "General Considerations," a temporary relaxation of the single failure criteria, consistent with overall reliability considerations, to allow limited time periods during which corrective actions may be taken to restore the AFW pumps to full operability. The proposed amendment to 15.3.4.C.2 serves to clarify that only one of the three operable AFW pumps associated with a single unit may be taken out of service at one time. This is consistent with the Bases and with TS 15.3.4.C.1 for two-unit operation. The proposed change requires no hardware or procedural change and can be characterized as administrative in nature. The staff finds this change to TS 15.3.4.C.2 to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards

consideration and there has been no public comment on such finding (57 FR 30264). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also involves changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Lombardo

Date: July 31, 1992