

February 8, 1989

Docket Nos. 50-266  
and 50-301

Mr. C. W. Fay, Vice President  
Nuclear Power Department  
Wisconsin Electric Power Company  
231 West Michigan Street, Room 308  
Milwaukee, Wisconsin 53201

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Dear Mr. Fay:

SUBJECT: AMENDMENT NOS. 116 AND 119 TO FACILITY OPERATING LICENSE NOS. DPR-24  
AND DPR-27 (TAC NOS. 68369 AND 68370)

The Commission has issued the enclosed Amendment Nos. 116 and 119 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated May 24, 1988 and supplemented on November 7, 1988.

These amendments revise the testing frequency specified in Technical Specification Table 15.4.1-1 for a number of instrumentation channels.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Warren H. Swenson, Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 116 to DPR-24
2. Amendment No. 119 to DPR-27
3. Safety Evaluation

cc w/enclosures:  
See next page

\*SEE PREVIOUS CONCURRENCE:

|          |            |            |            |            |
|----------|------------|------------|------------|------------|
| Office:  | LA/PDIII-3 | PM/PDIII-3 | PD/PDIII-3 | ICSB       |
| Surname: | *PKreutzer | *WSwenson  | *JHannon   | *SNewberry |
| Date:    | 1/5/89     | 1/10/89    | 1/10/89    | 1/13/89    |

*AB*  
OGC-WF1  
*R Bachmann*  
1/13/89

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*CC*

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Milwaukee, Wisconsin 53201

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cc w/enclosures:  
See next page

Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 1/15/89

PM/PDIII-3  
WSwenson  
1/10/89

PD/PDIII-3  
JHannon  
1/10/89

ICB  
SNewberry  
1/13/89

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| WSwenson     | GPA/PA     |
| JHannon      | ARM/LFMB   |
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Mr. C. W. Fay  
Wisconsin Electric Power Company

Point Beach Nuclear Plant  
Units 1 and 2

cc:

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U.S. Nuclear Regulatory Commission  
6612 Nuclear Road  
Two Rivers, Wisconsin 54241



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated May 24, 1988 and supplemented on November 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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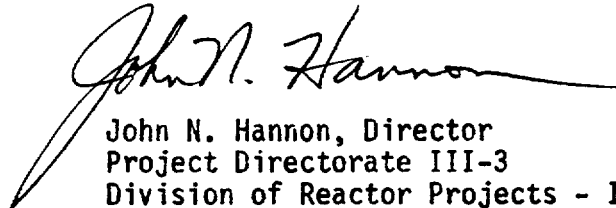
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 8, 1989



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated May 24, 1988 and supplemented on November 7, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

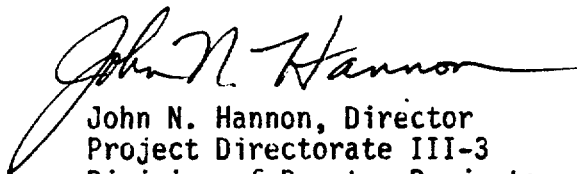
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 8, 1989

ATTACHMENT TO LICENSE AMENDMENT NOS. 116 AND 119  
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27  
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

Table 15.4.1-1 (1 of 4)  
Table 15.4.1-1 (2 of 4)

INSERT

Table 15.4.1-1 (1 of 4)  
Table 15.4.1-1 (2 of 4)



TABLE 15.4.1-1  
(Page 1 of 4)

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TEST OF INSTRUMENT CHANNELS

| No. | Channel Description         | Check                | Calibrate            | Test          | Remarks   |
|-----|-----------------------------|----------------------|----------------------|---------------|---|
| 1.  | Nuclear Power Range         | S(1)**<br>M*(3)**(4) | D(1)**<br>Q*(3)**(4) | M(2)**        | (1) Heat Balance<br>(2) Signal to $\Delta T$ ; bistable action (permissive, rod stop, trips)<br>(3) Upper and lower chambers for axial off-set<br>(4) Compare incore to excore axial flux difference. Recalibrate if the absolute difference is greater than or equal to 3 percent. |
| 2.  | Nuclear Intermediate Range  | S(1)**               | N.A.                 | P(2)          | (1) Once/shift when in service<br>(2) Log level; bistable action (permissive, rod stop, trips)  |
| 3.  | Nuclear Source Range        | S(1)                 | N.A.                 | P(2)          | (1) Once/shift when in service<br>(2) Bistable action (alarm, trips)  |
| 4.  | Reactor Coolant Temperature | S                    | R                    | M(1)**<br>(2) | (1) Overtemperature- $\Delta T$<br>(2) Overpower - $\Delta T$   |
| 5.  | Reactor Coolant Flow        | S**                  | R                    | M**           |   |
| 6.  | Pressurizer Water Level     | S**                  | R                    | M**           |   |
| 7.  | Pressurizer Pressure        | S**                  | R                    | M**           |   |
| 8.  | 4 Kv Voltage                | N.A.                 | R                    | M**           | Reactor protection circuits only  |
| 9.  | Analog Rod Position         | S(1)**               | R                    | M**           | (1) With step counters  |

\*By means of the movable in-core detector system.

\*\*Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period. Tests of permissive and low power trip bistable setpoints which cannot be done during power operations shall be conducted prior to startup if not done in the previous two weeks.

TABLE 15.4.1-1 (Continued)  
(Page 2 of 4)

| No. | Channel Description                | Check  | Calibrate | Test   | Remarks  |
|-----|------------------------------------|--------|-----------|--------|--|
| 10. | Rod Position Bank Counters         | S(1)** | N.A.      | N.A.   | (1) With analog rod position   |
| 11. | Steam Generator Level              | S**    | R         | M(1)** | (1) Includes test of logic for reactor trip on low-low level and automatic actuation logic for auxiliary feedwater pumps |
| 12. | Steam Generator Flow Mismatch      | S**    | R         | M**    |  |
| 13. | Charging Flow                      | N.A.   | R         | N.A.   |  |
| 14. | Residual Heat Removal Pump Flow    | N.A.   | R         | N.A.   |  |
| 15. | Boric Acid Tank Level              | D      | R         | N.A.   |  |
| 16. | Refueling Water Storage Tank Level | N.A.   | R         | N.A.   |  |
| 17. | Volume Control Tank Level          | N.A.   | R         | N.A.   |  |
| 18. | Reactor Containment Pressure       | S      | R         | M(1)** | (1) Isolation valve signal   |
| 19. | Radiation Monitoring System        | D      | R         | M      | (1) Radioactive Effluent Monitoring Instrumentation Surveillance Requirements are specified in 15.7.4.                   |
| 20. | Boric Acid Control                 | N.A.   | R         | N.A.   |  |
| 21. | Containment Water Level            | M      | R         | N.A.   |  |
| 22. | Turbine Overspeed Trip*            | N.A.   | R         | M(1)** | (1) Block trip   |
| 23. | Accumulator Level and Pressure     | S      | R         | N.A.   |  |

\*Overspeed Trip Mechanism, and Independent Turbine Speed Detection and Valve Trip System.

\*\*Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 116 AND 119 TO  
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY  
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated May 24, 1988 and supplemented on November 7, 1988, Wisconsin Electric Power Company (the licensee) submitted an application for amendments to the Point Beach Nuclear Plant, Unit Nos. 1 and 2 licenses. The proposed amendments would revise the testing frequency contained in Technical Specification Table 15.4.1-1, "Minimum Frequencies for Checks, Calibrations, and Test of Instrument Channels," from biweekly to monthly for the following instrumentation channels:

- (1) Nuclear power range.
- (2) Reactor coolant temperature.
- (3) Reactor containment pressure.

The proposed amendments would also change the frequency for performing the channel check for the reactor containment pressure to once per shift. The proposed testing and checking of these instrumentation channels is consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors.

2.0 EVALUATION

The licensee requested that the testing frequency associated with Technical Specification Table 15.4.1-1, Item 1, "Nuclear Power Range," be changed from biweekly to monthly. This change would reduce the chances of generating a spurious reactor trip during testing. The licensee also requested that the testing frequency associated with Table 15.4.1-1, Item 4, "Reactor Coolant Temperature," be changed from biweekly to monthly. Biweekly testing of these channels results in biweekly testing of the reactor trip breakers. Frequent testing of the reactor trip breakers may be non-productive and WCAP-11312, "WOG Technical Specification Subcommittee Reactor Trip Breaker Maintenance/Surveillance Optimization Program," recommends less frequent reactor trip breaker testing. Finally, the licensee requested that the testing frequency associated with Table 15.4.1-1, Item 18, "Reactor Containment Pressure," be changed from biweekly to monthly. To remain fully consistent with Standard Technical Specifications, the frequency for performing the channel check for reactor containment pressure would be revised to once per shift.

The staff has reviewed the licensee's submittals and concurs that the biweekly testing required for the nuclear power range, reactor coolant temperature, and containment pressure channel logic is beyond industry standards and is unduly restrictive. These channels have proven to be very reliable in practice. Based on the most severe drift rate experienced during biweekly testing, the channels should remain within Technical Specification setpoint limits between monthly testing. The revised testing frequency is consistent with the channel logic test frequencies for other channels which use similar components and are of comparable safety significance. The requested change will make the test frequency conform with current industry standards and Standard Technical Specifications. The change in the channel check frequency for reactor containment pressure from daily to once per shift is in the conservative direction. The staff finds the proposed changes acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes to an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Wagner and W. Swenson

Dated: February 8, 1989