

April 25, 1989

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

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Dear Mr. Fay:

SUBJECT: AMENDMENT NOS. 119 AND 122 TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27 (TACS 72226/72227)

The Commission has issued the enclosed Amendment Nos. 119 and 122 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments revise the Technical Specifications in response to your application dated February 24, 1989 and supplemented on March 3, 1989.

These amendments revise the permissible bypass conditions for item 3.b, "Auxiliary Feedwater," of Technical Specification Table 15.3.5-3, "Emergency Cooling."

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Warren H. Swenson, Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 119 to DPR-24
2. Amendment No. 122 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

Office:	LA/PDIII-3	PE/PDIII-3
Surname:	PKreutzer	LKokajko/mr
Date:	4/10/89	4/10/89

PM/PDIII-3
WSwenson
4/10/89

PD/PDIII-3
JHannon
4/10/89

OGC-WF1
RBachmann
4/13/89

for 4/25/89

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CP-1
cc

Mr. C. W. Fay
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Units 1 and 2

cc:

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Resident Inspector's Office
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated February 24, 1989 and supplemented on March 3, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.119 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas V. Wambach, Acting Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1989



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 122
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated February 24, 1989 and supplemented on March 3, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

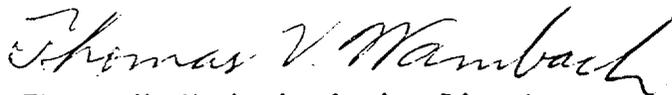
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.122 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas V. Wambach, Acting Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 25, 1989

ATTACHMENT TO LICENSE AMENDMENT NOS. 119 AND 122
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE
Table 15.3.5-3 (1 of 2)

INSERT
Table 15.3.5-3 (1 of 2)

TABLE 15.3.5-3

EMERGENCY COOLING

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>NO. OF</u> <u>CHANNELS</u>	<u>2</u> <u>NO. OF</u> <u>CHANNELS</u> <u>TO TRIP</u>	<u>3</u> <u>MINIMUM</u> <u>OPERABLE</u> <u>CHANNELS</u>	<u>4</u> <u>MINIMUM</u> <u>DEGREE OF</u> <u>REDUNDANCE</u>	<u>5</u> <u>PERMISSABLE</u> <u>BYPASS</u> <u>CONDITIONS</u>	<u>OPERATOR ACTION</u> <u>IF CONDITIONS OF</u> <u>COLUMN 3 OR 4</u> <u>CANNOT BE MET</u>
1.	SAFETY INJECTION						
a.	Manual	2	1	1	1		Hot Shutdown***
b.	High Containment Pressure	3	2	2	1		Hot Shutdown***
c.	Steam Generator Low Steam Pressure/Loop	3	2	2	1	Primary Pressure is Less than 1800 psig	Hot Shutdown***
d.	Pressurizer Low Pressure	3	2	2	1	Primary Pressure is Less than 1800 psig	Hot Shutdown***
2.	CONTAINMENT SPRAY						
a.	Manual	2	2	2	--**		Hot Shutdown***
b.	Hi-Hi Containment Pressure (Containment Spray)	2 sets of 3	2 of 3 in each set	2 per set	1/set		Hot Shutdown***
3.	AUXILIARY FEEDWATER						
a.	Low-Low Steam Generator Water						
i.	Start Motor Driven Pump	3/steam gen.	2 either gen.	2/steam gen.	1		Hot Shutdown***
ii.	Start Turbine Driven Pump	3/steam gen.	2/both gens.	2/steam gen.	1		Hot Shutdown***
b.	Trip of Both Main Feedpumps Starts Motor Driven Pumps*	2/pump	1/pump	1/pump	1	Reactor Power is Less than 40%	Hot Shutdown***
c.	Undervoltage on 4KV Busses Starts Turbine Driven Pump	2/bus	1/bus	1/bus	--		Hot Shutdown***



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 119 AND 122 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By letter dated February 24, 1989 and supplemented on March 3, 1989, Wisconsin Electric Power Company (the licensee) submitted an application for amendments to the Point Beach Nuclear Plant, Units Nos. 1 and 2 licenses. The proposed amendments will revise item 3.b, "Auxiliary Feedwater," of Technical Specification Table 15.3.5-3, "Emergency Cooling." This change is necessitated by the installation of the ATWS mitigating system actuation circuitry (AMSAC) at Point Beach, which will bypass the "Trip of both main feedpumps starts motor driven pump" auxiliary feedwater actuation when reactor power is less than 40%.

2.0 EVALUATION

The licensee requested the amendment to clarify the permissible bypass conditions of an existing trip circuit for the motor driven auxiliary feedwater pumps. The proposed amendments would add a permissible bypass condition for the start of the auxiliary feedwater motor driven pumps upon trip of both main feedwater pumps. This bypass would be an operational bypass, such that when reactor power is less than 40%, the automatic start of the auxiliary motor driven feedwater pumps would be bypassed upon trip of both main feedwater pumps. This change is based upon the design and installation of the AMSAC at Point Beach Nuclear Plant.

The licensee has provided to the NRC a description of the proposed AMSAC for installation at Point Beach Nuclear Plant. This information was provided in a letter dated April 23, 1987 and supplemented in letters dated December 30, 1987 and March 3, 1988. In these submittals, it was demonstrated that the AMSAC equipment would have the quality and reliability necessary to perform its intended function while minimizing the potential for transients that may challenge the existing plant safety systems. This is accomplished by the use of instrumentation and logic hardware which are separate and diverse from that used in the reactor trip system. The power supplies for AMSAC are also independent and diverse from the reactor protection system power supplies.

Provisions are included in the AMSAC design for maintenance and testability using output bypass switching. The licensee has committed to a complete end-to-end test of the system during each refueling outage and testing of

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those portions of AMSAC that can be tested at power, such as bistables, time delays, and logic relays, on a semi-annual basis. The testability provisions and periodic surveillance will provide assurance that the AMSAC is available to reliably perform its intended function.

The only safety related-interface of the AMSAC will be between the AMSAC and the auxiliary feedwater system. Isolation at this interface will be achieved using electrical isolation devices that have been qualified and tested to class 1E electrical equipment requirements. The licensee has concluded that the provisions in the AMSAC design and installation do not involve an unreviewed safety question, as defined in 10 CFR 50.59.

The Point Beach Nuclear Plant AMSAC design was approved in a safety evaluation by the Office of Nuclear Reactor Regulation, "Compliance with ATWS Rule 10 CFR 50.62," dated August 4, 1988. This design, as approved, provided for an operational bypass at 40% reactor power.

Since this is an administrative change to clarify an AMSAC-related requirement which is required under 10 CFR 50.62, and to enhance the technical specifications, the staff finds the proposed change acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence E. Kokajko

Dated: April 25, 1989