

December 3, 1987

Docket Nos. 50-266  
and 50-301

Mr. C. W. Fay, Vice President  
Nuclear Power Department  
Wisconsin Electric Power Company  
231 West Michigan Street, Room 308  
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

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The Commission has issued the enclosed Amendment Nos. 109 and 112 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated August 26, 1987.

These amendments incorporate a change to Technical Specification 15.4.11, "Control Room Emergency Filtration," and administrative changes to non-radiological Technical Specifications 16.1 and 16.5.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original Signed By:

David H. Wagner, Project Manager  
Project Directorate III-3  
Division of Reactor Projects

Enclosures:

1. Amendment No. 109 to DPR-24
2. Amendment No. 112 to DPR-27
3. Safety Evaluation

cc w/enclosures:

See next page

Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 11/3/87

PM/PDIII-3  
DWagner/r1  
11/3/87

SPLB  
JLee  
11/6/87

SPLB  
JKudrick  
11/9/87

Office: SPLB  
Surname: JCraig  
Date: 11/10/87

PD/PDIII-3  
DWigginton  
11/3/87

OGC  
Johnson  
11/1/87

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PDR ADDCK 05000266  
PDR

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OK as  
mined  
11-30-87

Mr. C. W. Fay  
Wisconsin Electric Power Company

Point Beach Nuclear Plant  
Units 1 and 2

cc:  
Mr. Bruce Churchill, Esq.  
Shaw, Pittman, Potts and Trowbridge  
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Washington, DC 20037

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Town Chairman  
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Chairman  
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Office of Executive Director  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 109  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated August 26, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated August 26, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 109, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David L. Wigginton, Acting Director  
Project Directorate III-3  
Division of Reactor Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 3, 1987

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David L. Wigginton, Acting Director  
Project Directorate III-3  
Division of Reactor Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 3, 1987

ATTACHMENT TO LICENSE AMENDMENT NOS. 109 AND 112  
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27  
DOCKET NOS. 50-266 AND 50-301

Revise the Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Appendix A

REMOVE

page 15.4.11-2

INSERT

page 15.4.11-2

Appendix B

page 16.1-1  
page 16.1-3

page 16.1-1  
page 16.1-3

- c. Halogenated hydrocarbon testing of the charcoal adsorber bank shall be performed after each complete or partial replacement of charcoal adsorbers or after any structural maintenance of the adsorber housing. Halogenated hydrocarbon testing shall be at design velocity  $\pm 20\%$ .
- d. Laboratory sample analysis of in-place charcoal adsorbent shall be performed at least once per year for standby service or after every 720 hours of system operation and, as a minimum, shall be conducted at velocities within 20% of design, 1.75 mg/m<sup>3</sup> inlet iodide concentration, 95% relative humidity and 30°C (86°F).
- e. Fans shall be tested at least once per year or after 720 hours of operation since the previous test, and following fan maintenance or repair.

#### Basis

The control room emergency filtration system is designed to filter the control room atmosphere and makeup air to the control room during control room isolation conditions. The control room emergency filtration is normally isolated and not in operation and testing more frequently than that specified is not required to insure operability or performance. If the efficiencies of HEPA and charcoal adsorbers are as specified, the resulting control room doses during accident conditions will be less than allowable levels in Criterion 19 of Appendix A to 10 CFR 50.



## 16. NONRADIOLOGICAL TECHNICAL SPECIFICATIONS

### 16.1 Definitions

The definitions for terms used in these Nonradiological Technical Specifications for Units 1 and 2 are stated below.

#### WPDES Permit

The WPDES permit is the Wisconsin Pollutant Discharge Elimination System Permit No. WI-0000957 issued by the State of Wisconsin Department of Natural Resources for the Wisconsin Electric Power Company, Point Beach Nuclear Plant, and as subsequently amended.

## 16.5 Reporting Requirements

### Specification

1. As part of the Semiannual Monitoring Report, described in Section 15.7.8.4.A of Appendix A, the following shall be reported:
  - a. All scheduled and unscheduled chemical discharge to the condenser cooling water.
  - b. A description of circulating water system operation for each unit which includes ambient temperature, intake temperature, discharge temperature, and circulating water system flow.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 10 AND 11 TO  
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY  
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

In a letter dated August 26, 1987, Wisconsin Electric Power Company (the licensee) submitted an application for amendment of the Point Beach 1 and 2 licenses. The purpose of the amendments is to incorporate a change to Technical Specification 15.4.11, "Control Room Emergency Filtration", and to make administrative changes to Technical Specifications 16.1 and 16.5. Specifically, the proposed change to Technical Specification 15.4.11 modifies the laboratory sample analysis parameters required for the periodic iodine-removal efficiency testing of in-place charcoal adsorbent. The temperature and iodide concentration parameters for this analysis are currently 125°F and 0.05 milligrams of I-131 labeled CH<sub>3</sub>I per cubic meter of air (mg/m<sup>3</sup>), respectively. The proposed changes to the Technical Specifications would modify those values to 30°C (86°F) and 1.75 mg/m<sup>3</sup>, respectively. The proposed change to Technical Specification 16.1, "Definitions", would revise the wording of the definition of "WPDES Permit". The definition currently identifies the permit number, including the permit revision number, and the date of revision. The proposed Technical Specification change would revise the wording to eliminate the revision number and date, but will retain the wording ". . . and as subsequently amended" to imply the most recent permit revision. Finally, Technical Specification 16.5, "Reporting Requirements", would be revised to correct an improper reference to another part of the Technical Specifications.

2.0 EVALUATION

As part of the amendment requests submitted by the licensee, the licensee proposed modifying the laboratory sample analysis parameters specified for the periodic iodine-removal efficiency testing of charcoal adsorbent specified in Technical Specification 15.4.11. Specifically, the current temperature and iodide concentrations of 125°F and 0.05 mg/m<sup>3</sup>, respectively, specified in the technical specification would be revised to 30°C (86°F) and 1.75 mg/m<sup>3</sup>, respectively.

On August 28, 1986, the NRC Office of Inspection and Enforcement issued IE Information Notice 86-76, "Problems Noted in Control Room Emergency Ventilation Systems". An attachment to that Notice, "Summary of Control Room Habitability Reviews", lists common problems noted in control room emergency ventilation systems during NRC plant visits to gather information on control room habitability. One problem noted was that laboratory testing of charcoal adsorber efficiency is being performed at temperatures much higher than any temperature expected during the course of an accident, resulting in erroneously high efficiency measurements, as compared to testing conducted at 30°C.

The licensee's proposed change would adopt 30°C as the specified temperature parameter and 1.75 mg/m<sup>3</sup> as the specified iodide concentration under which laboratory testing would be done. This temperature and concentration confirm ASTM Standard D3803-79, "Radioiodine Testing of Nuclear-Grade Gas-Phase Adsorbents", as the standard method for testing iodine-removal efficiency of activated charcoal beds. This temperature and concentration are specified for the test method measuring iodine removal under normal plant operating conditions. While the purpose of the emergency filtration system is to provide a safe control room atmosphere in a post-design basis accident environment, that environment will not be different in terms of temperature from normal conditions. Additionally, raising the iodide concentration from .05 mg/m<sup>3</sup> to 1.75 mg/m<sup>3</sup> will tend to make the testing conservative, because at higher iodide concentrations, the probability that the challenge gas will leak past the filter being tested is increased. This results in a reduced calculated efficiency for the tested filter. Standard Review Plan Section 6.5.1, "ESF Atmosphere Cleanup Systems", and Regulatory Guide 1.52 delineate that laboratory testing of activated carbon adsorbent should conform with ANSI N509-1975 or ANSI N509-1980. These ANSI Standards, in turn specify ASTM-3803-79 as the acceptable standard method. Therefore, the licensee's proposed modification of the temperature parameter from 125°F to 30°C (86°F) and the concentration parameter from .05 mg/m<sup>3</sup> to 1.75 mg/m<sup>3</sup> is acceptable.

The licensee also proposed administrative changes to two non-radiological Technical Specifications. The licensee proposed that specification 16.1, "Definitions", be revised to include a more generic wording of the definition of "WPDES Permit". Presently, the definition spells out the permit number, including the permit revision number and date of revision. The licensee's proposed rewording will eliminate the revision number and date, but will retain the wording "... and as subsequently amended" to imply the most current permit revision. The change would not require that the Technical Specifications be changed whenever a new revision of the WPDES permit is issued. Additionally, this change will not relieve the licensee from its previous commitment to report changes to its WPDES permit. This proposed change is administrative in nature, and is acceptable.

Finally, the licensee proposed that specification 16.5 be modified to correct an improper reference to another part of the Technical Specifications. The specification currently refers to Section 15.6.9.2.C of Appendix A, whereas the correct reference should be to Section 15.7.8.4.A of Appendix A. This proposed change is administrative in nature, and is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and in a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). These amendments also involve changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22 (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 3, 1987

Principal Contributor: David H. Wagner, NRR