

April 14, 1988

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

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Dear Mr. Fay:

The Commission has issued the enclosed Amendment Nos. 113 and 116 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated October 13, 1987.

These amendments revise the asterisked notes to Technical Specification Table 15.4.1-1, Item 25, "Steam Generator Pressure," which clarifies the requirements for testing of the steam generator pressure channels during refueling shutdowns.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

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David H. Wagner, Project Manager
Project Directorate III-3
Division of Reactor Projects

Enclosures:

1. Amendment No. 113 to DPR-24
2. Amendment No. 116 to DPR-27
3. Safety Evaluation

cc w/enclosures:

See next page

Office: LA/PDIII-3
Surname: PKreutzer
Date: 3/16/88

PM/PDIII-3
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Mr. C. W. Fay
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Point Beach Nuclear Plant
Units 1 and 2

cc:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 113
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 13, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 113, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 13, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 116, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance. The Technical Specifications are to be implemented within 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1988

ATTACHMENT TO LICENSE AMENDMENT NOS.113 AND 116
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

Table 15.4.1-1 (pg. 3)
Table 15.4.1-1 (pg. 4)

INSERT

Table 15.4.1-1 (pg. 3)
Table 15.4.1-1 (pg. 4)

TABLE 15.4.1-1 (3 of 4)

<u>No.</u>	<u>Channel Description</u>	<u>Check</u>	<u>Calibration</u>	<u>Test</u>	<u>Remarks</u>
24.	Containment Pressure	S	R	M**	Narrow range containment pressure (-3.0, +3 psig excluded)
25.	Steam Generator Pressure	S***	R	M**	
26.	Turbine First Stage Pressure	S**	R	M**	
27.	Emergency Plan Radiation Survey Instruments	Q	R	Q	
28.	Environmental Monitors	M	N.A.	N.A.	
29.	Overpressure Mitigating	S	R	****	
30.	PORV Position Indicator	S	R	R	
31.	PORV Block Valve Position Indicator	Q	R	N.A.	
32.	Safety Valve Position Indicator	M	R	N.A.	
33.	PORV Operability	N.A.	R	M	Performance of a channel functional test but excluding valve operation.
34.	Subcooling Margin Monitor	M	R	N.A.	
35.	Undervoltage on 4 KV Bus	N.A.	R	M**	For Auxiliary Feedwater Pump Initiation
36.	Auxiliary Feedwater Flow Rate	See Remarks	R	N.A.	Flow Rate indication will be checked at each unit startup and shutdown.
37.	Degraded 4.16 KV Voltage	S	R	M**	
38.	a. Loss of Voltage (4.16 KV)	S	R	M**	
	b. Loss of Voltage (480 V)	S	R	M**	
39.	4160 V Frequency	N.A.	R	N.A.	

Unit 1 - Amendment No. 38, 47, 55, 59, 113
Unit 2 - Amendment No. 30, 33, 30, 34, 116

TABLE 15.4.1-1 (Page 4 of 4)

<u>No.</u>	<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
40.	Containment High Range Radiation	S **	R	M **	Calibration to be verification of response to a source.
41.	Containment Hydrogen Monitor	D	R/Q	N.A.	Gas Calibration - Q, Electronic Calibration - R Sample gas for calibration at 2% and 6% hydrogen.
42.	Reactor Vessel Fluid Level System	M	R	N.A.	
43.	In-Core Thermocouple	M	R	N.A.	Calibration to be verification of response to a source.

S - Each Shift
 D - Daily
 W - Weekly
 B/W - Biweekly
 Q - Quarterly

M - Monthly
 P - Prior to each startup if not done previous week.
 R - Each Refueling interval (But not to exceed 18 months).
 N.A. - Not applicable.

**Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.

***During cold or refueling shutdown, a check of one pressure channel per steam generator is required when the steam generator could be pressurized.

****When used for the overpressure mitigating system each PORV shall be demonstrated operable by:

- a. Performance of a channel functional test on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required operable and at least once per 31 days thereafter when the PORV is required operable.
- b. Testing valve operation in accordance with the inservice test requirements of the ASME Boiler and Pressure Vessel Code, Section XI.

Unit 1 - Amendment No. 38,47,55,76,92, 113
 Unit 2 - Amendment No. 50,55,60,80,96, 116



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 113 AND 116 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

INTRODUCTION

In a letter dated October 13, 1987, Wisconsin Electric Power Company (the licensee) submitted an application for amendments of the Point Beach, Units 1 and 2, Technical Specifications (TS). The proposed amendments would revise Technical Specification Table 15.4.1-1, "Minimum Frequencies for Checks, Calibrations and Test of Instrument Channels," Item 25, "Steam Generator Pressure," surveillance requirements. This TS identifies the requirements for the checking, calibrating, and testing of steam generator pressure channels. Currently, TS Table 15.4.1-1, Item 25, requires, in part:

- (1) that all three steam generator pressure channels (per steam generator) be checked on a shift basis, except during periods of refueling shutdowns if the steam generator vessel temperature is greater than 70°F, and
- (2) that a steam generator pressure channel test be performed on a monthly basis, except during periods of refueling shutdown if the steam generator vessel temperature is greater than 70°F.

In its October 13, 1987 letter, the licensee proposed revising these requirements, respectively, to require:

- (1) that all three steam generator pressure channels be checked on a shift basis, except during cold shutdown or refueling shutdown, when a check of one pressure channel per steam generator is required if the steam generator could be pressurized, and
- (2) that a steam generator pressure channel test be performed on a monthly basis, except during periods of refueling shutdown; however, the test must be performed prior to startup if it has not been performed during the previous surveillance period.

The changes proposed by the licensee do not result in changes in the frequency of the required surveillances during power operation. Instead, the proposed changes only affect the frequency of required surveillances during shutdowns, provided certain conditions are met. The actual amendment is not a revision to the surveillance frequencies, but a revision to the notes identified by asterisks for the steam generator pressure channel check and test.

EVALUATION

Technical Specification 15.3.1.B.2, "Pressure Temperature Limits," requires that the steam generators not be pressurized above 200 psig whenever the steam generator vessel temperature is less than 70°F. The intent of this TS is to ensure that pressure induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The surveillance specified in TS Table 15.4.1-1, Item 25 details the surveillance requirements applicable to the steam generator pressure channels. This surveillance is in place to (1) ensure that the plants are operated in conformance with TS 15.3.1.B.2 and (2) verify that the safety injection actuation alarm and trip initiation are received when a test signal is injected through the steam generator pressure channels.

The licensee noted two problems with the current TS. First, the steam generator vessels are not instrumented for temperature; therefore, it is not practical for operators to determine when the steam generator vessel temperature is less than 70°F. Second, the currently required monthly channel test only verifies the steam generator pressure channels are operable by injecting a test signal which verifies that the alarm and trip functions of safety injection actuation are operable. When the plant is in the cold shutdown or refueling shutdown mode, the low-pressure alarm is in constant alarm (steam generator pressure is less than the alarm setpoint) and automatic safety injection is not necessary (the plant is in cold shutdown or refueling shutdown). The licensee states that since the monthly test does not improve the operator's means of monitoring the steam generator pressure during periods when the limits of specification 15.3.1.B.2 could be violated, the test is useless.

The staff has reviewed the changes proposed by the licensee and concludes that these changes are acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Wagner

Dated: April 14, 1988